

## HIGH POWER ICRH AND NB HEATING RESULTS IN TEXTOR

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### ABSTRACT

Neutral beam injection (NBI-co and NBI-counter), ICRH and their combinations have been compared on TEXTOR for what concerns the heating and their effect on the energy anisotropy and the confinement. In most of the cases, stationary heated plasma conditions have been obtained with boronized wall. The main results are :

- (i) Production of a hot ion mode with NBI-co. It is characterized by a large energy anisotropy, by a large non inductively driven toroidal current and by the stabilization of the sawteeth (of the monster type) due to the hot ion tail.
- (ii) Significant enhancement of these effects when ICRH is added to NBI-co. The neutron yield is also increased by the addition of ICRH. Peak electron and ion temperature around 3 keV have been obtained and 70 % of the total current has been non inductively driven.
- (iii) Large excess of the total energy content (up to 2.3) with respect to the L-mode scaling predictions has been obtained with the combinations NBI-co + ICRH or NBI-co + NBI-counter.
- (iv) Up to 6 MW of additional power has been coupled to the plasma leading to  $\beta_p = 1.5$  and  $\beta_t$  equal to 70 % of the Troyon limit.

### KEYWORDS

Co-and counter-neutral beam injection - ICRH - combined heating - confinement - sawtooth stabilization - monster sawteeth - non inductively driven current.

### 1. INTRODUCTION

Auxiliary heating of TEXTOR has been carried out by means of co- and counter-beam injection and of ICRH. These heating methods and their combinations have been carefully compared.

During the reported experiments TEXTOR has been operated in the following conditions : plasma current  $I_p = 340$  kA (except otherwise stated), toroidal field  $B_t = 2.25$  T, small and large

plasma radii respectively of  $a = 46$  cm and  $R = 175$  cm with a plasma circular in shape and limited by the toroidal belt limiter ALT-II. The neutral beam system consists of one co - and one counter-beam line (Euringer *et al.*, 1989), each equipped with one modified ion source of the type "JET-PINI", and for the present experiments, is injecting deuterium into a deuterium plasma, tangentially to the major radius  $R$ , with a beam voltage of 50-55 kV, a total power up to  $2 \times 1.7$  MW and a beam divergence less than 1 degree. The beam power is split between the components at full, half and third energy respectively in the ratio  $\sim 69, 24$  and  $7\%$ . The ICRH set up consists of two independent heating lines (Messiaen *et al.*, 1989a) each fed by one 2 MW generator operated at 32.5 MHz and connected to a pair of  $\lambda/7$  antennae. Each pair of antennae is fed in phase opposition and provides a low field side RF excitation. Pure ICRH minority heating is used as heating scenario in a D-(H) plasma with a hydrogen concentration lower than  $10\%$ . Second harmonic interaction with the ions of the beam cannot be excluded in the case of combined NBI - ICRH.

The present experiments are performed with boronized wall conditioning (described by Winter *et al.*, 1989). Such a wall coating process allows operation with negligible contribution of the metallic impurities to the total radiation in presence of additional heating. It allows also stationary low impurity conditions and the minimization or avoidance of the density increase during the heating pulse. With ICRH, the use of antenna pairs fed in phase opposition further decreases the interaction with the wall and does not lead to a drop in plasma loading with respect to the in-phase feeding (Messiaen *et al.*, 1989a, Weynants *et al.*, 1989a). This has allowed us to couple in stationary conditions up to  $90\%$  of the nominal power of the RF generators (i.e. 3.6 MW) with radiation losses remaining below 600 kW and a density controlled by feedback. The boronized wall condition also enables to obtain a stationary hot ion mode with co-injection (NBI-co) at low density. With combined NBI + ICRH heating an additional power up to 6 MW has been presently coupled to the plasma and the total plasma energy content (including the contribution of the tails) has reached 145 kJ at  $I_p = 340$  kA and 180 kJ at  $I_p = 470$  kA leading to a maximum value of  $\beta_p = 1.5$  at 340 kA and 1.6 at 220 kA.

The main characteristics of each heating method and of their combinations will be discussed in the next sections. A first account on the NBI performances has been presented (Conrads *et al.*, 1989). Detailed analysis of the ICRH results with the present low field side excitation has been reported (Messiaen *et al.*, 1989a and Weynants *et al.* 1989a,b), and the first results of the combined operation NBI + ICRH are summarized in Messiaen *et al.*, 1989b.

## 2. NBI - CO HEATING

From one operation day to another, NBI-co leads to a large scatter in the plasma energy content, for apparently the same machine conditions. Lower points have in about equal proportion a lower ion and electron energy content as well as a lower tail contribution. It is not yet clear how to reach the best energy conditions but it appears to be a prerequisite to have low recycling conditions and optimal positioning of the plasma. On Fig. 1 the extent of the scatter of the diamagnetic energy is shown. In the following, we will concentrate our analysis on results pertaining to a set of data of the best regime obtained in successive discharges.

NBI-co heating leads to strong non-thermal components when the central line averaged density  $\bar{n}_{e0}$  is decreased. A typical shot description is given in Fig. 2 where the time evolution of the diamagnetic energy  $E_{dia}$ , the equilibrium (or MHD) energy  $E_{equi}$ , the central electron temperature  $T_{e0}$  (from ECE), the ion temperature  $T_{i,19.5}$  (measured 19.5 cm off axis, i.e. at  $R = 1.945$  m, by charge exchange spectroscopy), the loop voltage  $V_L$ , the density  $\bar{n}_{e0}$  (from HCN interferometer) and the total radiated power  $P_{rad}$  (from bolometry) are shown during a NBI pulse. The apparent stationarity of the plasma parameters, the large energy anisotropy, large central ion and electron temperatures, large loop voltage drop, and long sawtooth period are peculiar to the co-injection in TEXTOR. What is also apparent is the low ratio of  $P_{rad}$  to the total injected power  $P_{tot}$  ( $\sim 0.25$ ).

(i) Energy and temperature. The behaviour of  $E_{dia}$  and  $E_{equi}$  versus the density is shown in Fig. 3a. The electron contribution to the thermal component of the energy,  $E_{kin}$ , is computed from the HCN interferometer density profile  $n_e(r)$  and the temperature profile  $T_e(r)$  obtained from 9 ECE channels. The central temperature of ohmic plasmas is obtained from the neutron yield (assuming a Maxwellian distribution of the ions and an ion temperature profile homothetic to the electron one; the effect of  $Z_{eff}$  is also taken into account). The ionic energy component in auxiliary heated plasmas is

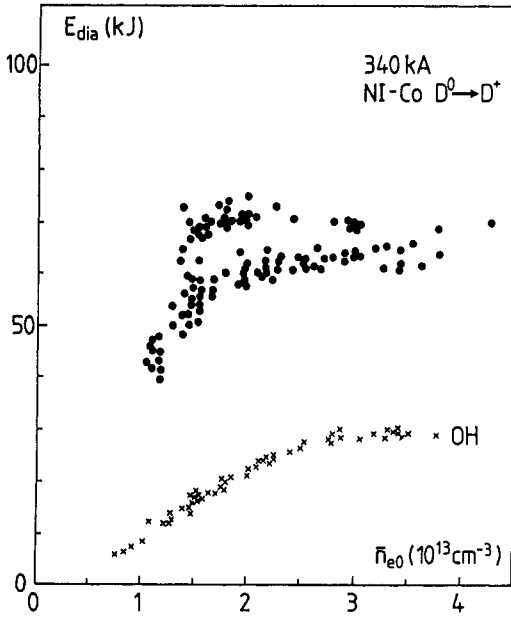


Fig. 1  $E_{dia}$  versus  $n_{eo}$  for NBI-co heated shots (data of many operational days). The OH values are also indicated.

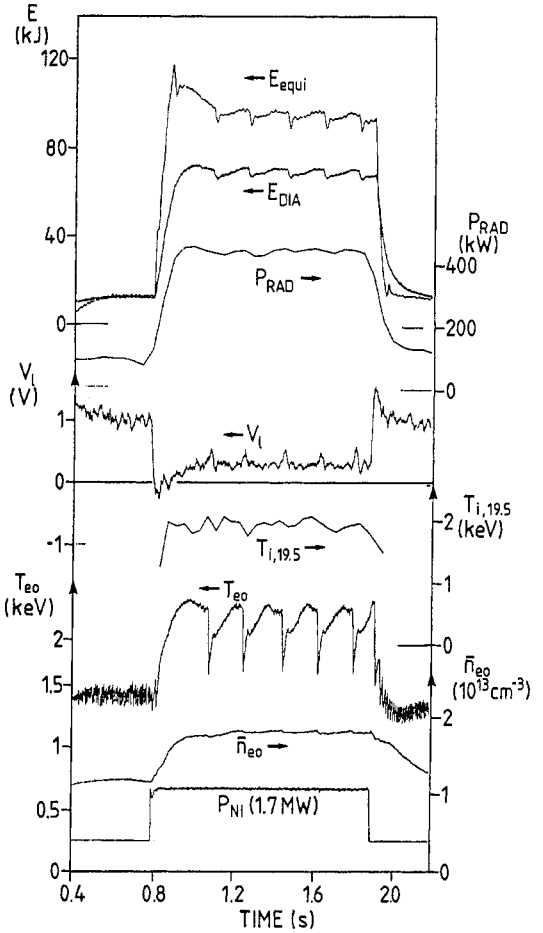


Fig. 2 Time evolution of  $E_{equi}$ ,  $E_{dia}$ ,  $P_{rad}$ ,  $V_l$ ,  $T_{l,19.5}$ ,  $T_{eo}$ ,  $n_{eo}$ ,  $P_{NI}$  during a NBI-co heated shot (# 37767).

rather poorly known, as the ion temperature is only measured with CXES in the presence of the co-beam and at one radial location. The actual magnetic surface on which this temperature prevails is found from the measured position of the magnetic axis. The ion temperature profile  $T_i(r)$ , and thus also the central value  $T_{i0}$ , is computed by the code TRANSP of Princeton (Hawryluk *et al.*, 1980) assuming neoclassical ion thermal diffusivity with a, radially constant, enhancement factor whose value is obtained by imposing the computed  $T_i(r)$  to pass through the measured point. The ion contribution to  $E_{kin}$  is then computed by TRANSP correcting the ion density for impurities and beam particles. The agreement between the three energy values for ohmic discharges and for heated ones, at sufficiently large plasma density, is usually better than 10 %. Large differences develop however because of strong non-thermal components in NBI heated discharges at low density, as shown in Fig. 3a. The distribution of the tail energy among its parallel and perpendicular components  $E_{//a}$  and  $E_{\perp a}$  can be obtained from  $E_{dia}$ ,  $E_{equi}$  and  $E_{kin}$ . The values thus obtained are confirmed by the predictions of TRANSP. On the other hand, note that the  $E_{kin}$  values can roughly be retrieved from the corresponding  $E_{equi}$  and  $E_{dia}$  values assuming the relation  $E_{//a} \approx 2 E_{\perp a}$ , predicted by the TRANSP simulations. In Fig. 3a we also show : (i) the total energy content  $E_{tot} = E_{kin} + E_{//a} + E_{\perp a} = (2 E_{equi} + E_{dia})/3$  which includes the tail contribution and the thermal component of the energy  $E_{kin}$  and (ii) the prediction of the original scaling of Kaye and Goldston, 1985, which is noted K-G. For the computation of the total power  $P_{tot}$ , we have used the sum of the engineering value of the injected NBI power  $P_{NI}$ , without taking into account the charge exchange losses, and the remaining ohmic power  $P_{OH} = V_l I_p$ . At low density,  $E_{kin}$  is on the K-G scaling but the total energy  $E_{tot}$  is a factor 2.4

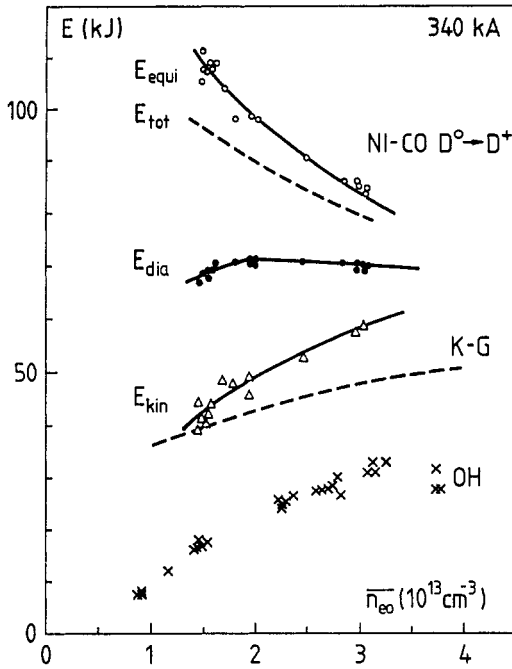


Fig. 3a Density scan of  $E_{\text{equi}}$ ,  $E_{\text{tot}}$ ,  $E_{\text{dia}}$  and  $E_{\text{kin}}$  during NBI-co. The corresponding predictions of the Kaye-Goldston scaling (noted K-G) and the OH values are also indicated.

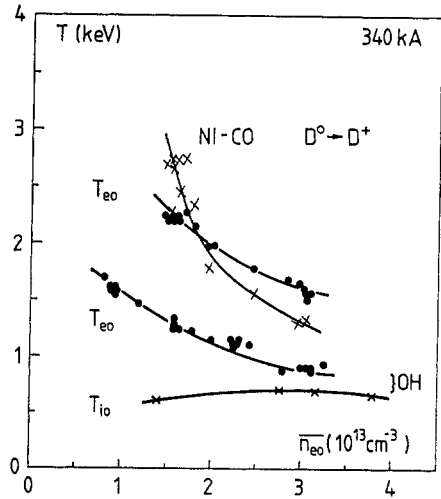


Fig. 3b  $T_{\text{io}}$  and  $T_{\text{eo}}$  values corresponding to Fig. 3a.

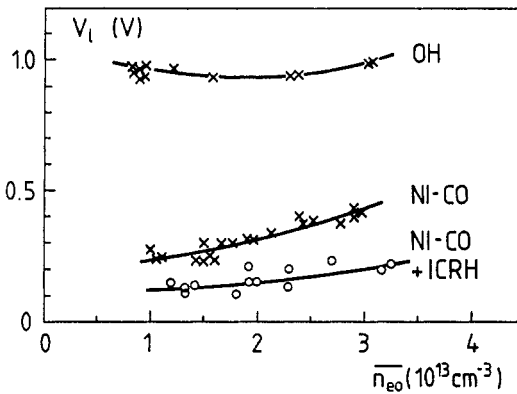


Fig. 3c  $V_l$  evolution corresponding to Fig. 3a. The values obtained for combined heating NBI-co + ICRH ( $1.5 < P_{\text{RF}} < 1.7$  MW) are also given.

above it, with an energy in the tails ( $E_{\text{tot}} - E_{\text{kin}}$ ) which is larger than the thermal component  $E_{\text{kin}}$ . The fact that  $E_{\text{dia}}$  is above the Kaye-Goldston scaling is due to the existence of a perpendicular tail. In Fig. 3b are shown the central ion temperature  $T_{\text{io}}$  computed as explained above, together with  $T_{\text{eo}}$ ; at a density  $\bar{n}_{\text{eo}} = 1.5 \times 10^{13} \text{ cm}^{-3}$ ,  $T_{\text{eo}}$  is above 2 keV and  $T_{\text{io}}$  reaches 2.6 keV. The ohmic values of  $T_{\text{eo}}$  and  $T_{\text{io}}$  are also given for completeness in Fig. 3b.

(ii) Loop voltage. The loop voltage evolution versus density in ohmic condition and with NBI-co heating is shown in Fig. 3c. The large loop voltage drop due to NBI heating can only be partially explained by the decrease of resistivity. An evaluation of the total non-ohmically driven

current  $I_{tD}$  (due to beam driven and bootstrap current) is obtained by

$$I_{tD} = I_p - \frac{V_\ell}{2\pi R} \int \sigma_{NC} dA \quad (1)$$

where  $\sigma_{NC}$  is the neoclassical resistivity and  $A$  the plasma cross section. The current  $I_{tD}$  varies from 180 kA to 110 kA when  $\pi_{e0}$  increases from  $1.4$  to  $3 \times 10^{13} \text{ cm}^{-3}$ . At low density, more than half of the plasma current is non-ohmically driven. Simulations by TRANSP of these shots have been done with as input : the NBI characteristics, the profiles  $N_e(r)$ ,  $T_e(r)$  and the  $Z_{eff}$  value, the ion temperature profile obtained as explained in the preceding paragraph. These simulations predict the values of  $V_\ell$ , resulting from the plasma resistivity and the non-ohmically driven current, which are in good agreement with the experimental ones and show that the contribution of the bootstrap current to  $I_{tD}$  lies between 35 and 40 kA.

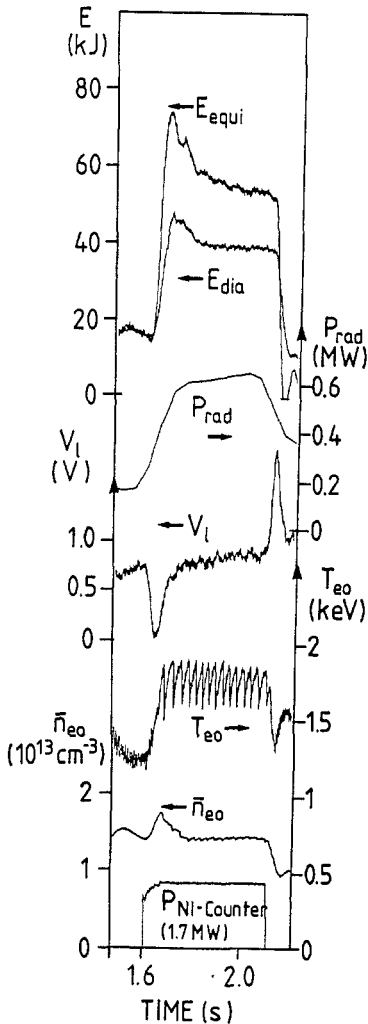


Fig. 4 Same data as in Fig. 2 but for a NBI-counter heated shot (# 41096).

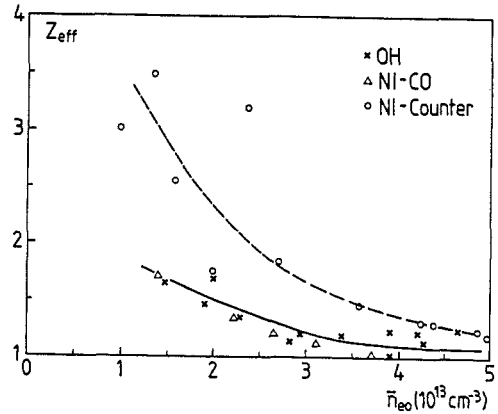


Fig. 5  $Z_{eff}$  evolution versus  $\pi_{e0}$  for OH, NBI-co and NBI-counter conditions as measured by soft X rays.

(iii) Sawtooth stabilization. NBI-co heating is very efficient in stretching the sawtooth period as seen in Fig. 2. For a given value of  $P_{N1}$  the period increases monotonically when the density decreases and below a threshold density ( $\sim 1.5 \times 10^{13} \text{ cm}^{-3}$  for the neutral beam conditions of Fig. 2) the sawteeth are completely stabilized,  $T_e$  remaining at its maximum value. A detailed analysis (Ongena *et al.*, 1990) shows that the stabilized sawtooth is of the "monster" type with a central  $q$  below 1 and that the results can be interpreted in terms of stabilization of the resistive internal  $m = 1$  mode by trapped energetic ions. The creation of trapped hot ion populations strongly peaked inside the  $q = 1$  surface appears to be essential for the stabilization. Note that sawtooth stabilization with NBI has also been observed on ASDEX in divertor discharges (Söldner *et al.*, 1986).

### 3. NBI-COUNTER HEATING AND COMPARISON WITH CO-INJECTION

A typical example of the time evolution of the plasma parameters during a NBI-counter pulse is given in Fig. 4 for a density comparable to that of the NBI-co shot of Fig. 2. It can be immediately appreciated that : (i) the difference between  $E_{dia}$  and  $E_{equi}$  is much smaller than for NBI-co and that the electron heating is also lower; (ii) the stationary loop voltage remains high; (iii) the sawtooth stretching is much smaller than for NBI-co.

The scatter of the energy measurements between the different days of operation is lower for counter-injection than for co-injection. The density increase produced by the NBI pulse is generally larger and higher values of  $Z_{eff}$  are obtained. This is shown in Fig. 5 where the measurement of  $Z_{eff}$  from soft X ray analysis for ohmic, co and counter discharges is displayed versus the density. When comparing the density scans of the energy shown on Fig. 3a and Fig. 6a, it appears that all the energies, including the tail energies, are lower for counter-injection. As there is no  $T_i$  measurement presently available in case of counter-injection, the value of  $E_{kin}$  is computed from  $E_{dia}$  and  $E_{equi}$  assuming the relation  $E_{i/a} = 2 E_{\perp a}$  which is also verified for the NBI-counter shot simulations by

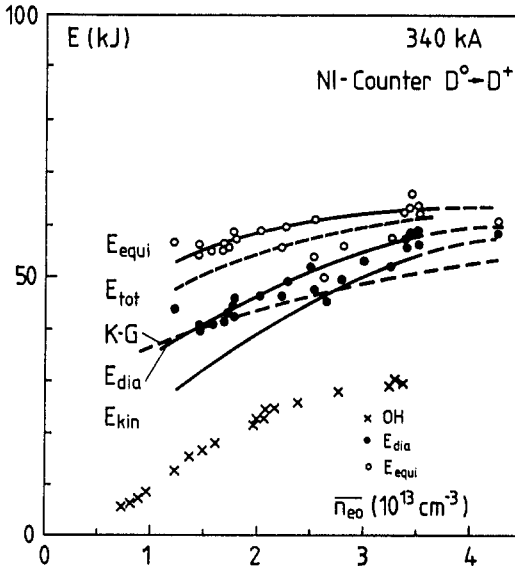


Fig. 6a Similar density scan as in Fig. 3a but for NBI-counter.

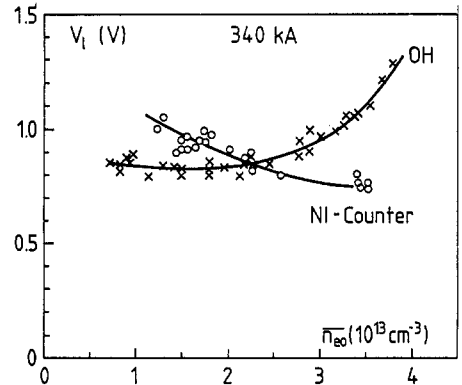


Fig. 6b  $V_L$  versus  $\pi_{eo}$  corresponding to the conditions of Fig. 6a.

TRANSP. At  $\pi_{eo} = 2 \times 10^{13} \text{ cm}^{-3}$ ,  $E_{tot}$  is only 62 % and  $E_{kin}$  78 % of the corresponding value for the best co-injection conditions. At low density  $E_{kin}$  is lower than the K-G scaling. The lower heating efficiency is also confirmed by the lower increase of the central electron temperature observed with NBI-counter heating as shown in Fig. 12. At sufficiently large density the heating performance of NBI-counter is approaching that of NBI-co.

The loop voltage behaviour is shown in Fig. 6b. At low density the loop voltage can be even higher than the ohmic value during the NBI-counter pulse. Now the beam driven current is opposed to the ohmic one thus cancelling the loop voltage drop due to the electron heating and an enhanced bootstrap current.

A part of this increase of  $V_L$  is also due to the rise of  $Z_{\text{eff}}$  shown in Fig. 5. TRANSP simulation of the shot shown in Fig. 4 leads to a value of  $I_{\text{TD}} = -60$  kA from which -87 kA are due to beam driven current and 27 kA to bootstrap current.

It is shown that the much lower capability of counter-injection as compared to co-injection in stretching the sawtooth period can be understood by the differences in magnitude and in spatial distribution of the trapped hot ion population inside the  $q = 1$  surface (Ongena *et al.*, 1990). It appears also that, opposite to the NBI-co case, the sawtooth period observed with NBI-counter is roughly density independent and that no complete sawtooth stabilization is observed.

#### 4. ICRF HEATING AND COMPARISON WITH NBI

The ICRH heating results pertaining to the minority heating in a (H)-D plasma with the present antenna system have been described and compared to theory (Beuken *et al.*, 1988 and Weynants *et al.*,

1989a and b). The results of a power scan at fixed density  $n_{e0} = 3 \times 10^{13} \text{ cm}^{-3}$ , corresponding to the best performances achieved up to now on TEXTOR, are given in Fig. 7a for the energies, in Fig. 7b for the central temperatures and in Fig. 7c for the loop voltage.  $T_{e0}$  is obtained from ECE and  $T_{i0}$  from the neutron yield. The values of  $T_{e0}$  are the same for NBI-co and ICRH and are lower for NBI-counter. The saturation appearing on the  $T_{e0}$  and  $T_{i0}$  values with ICRH at high  $P_{\text{tot}}$  is correlated with density profile broadening. The energies and central electron temperatures corresponding to the best performances of NBI-co and NBI-counter are also shown for comparison in Fig. 7a and b. The difference  $E_{\text{dia}} - E_{\text{equi}} = 3/4 (E_{\perp a} - 2E_{\parallel a})$ , where  $E_{\perp a}$  and  $E_{\parallel a}$  are respectively the energies of the perpendicular and parallel tails, is a measure of the tail anisotropy. For ICRH, the tail is almost completely perpendicular and  $E_{\text{dia}} > E_{\text{equi}}$ ; for NBI-co and NBI-counter we have, as stated before,

$E_{\parallel}/E_{\perp} \approx 2$  and consequently  $E_{\text{equi}} > E_{\text{dia}}$ . In table 1 we give, for the same  $n_{e0}$  and  $I_p$  as for Fig. 7, the values of the incremental confinement times  $\tau_{\text{inc,tot}}$  and  $\tau_{\text{inc,kin}}$  corresponding respectively to  $E_{\text{tot}}$  and  $E_{\text{kin}}$ . We quote also the electron and ion heating efficiencies  $\eta_{\alpha} = n_{e0} \Delta T_{\alpha} / P_{\text{aux}}$ ; where  $\alpha$  stands for electrons or ions. As for the NBI power  $P_{\text{NI}}$ , we use for the ICRH power  $P_{\text{RF}}$  the engineering value of the power, i.e. the power radiated into the tokamak, to compute  $P_{\text{tot}}$  and  $P_{\text{aux}} = P_{\text{NI}} + P_{\text{RF}}$ .

Table 1 :  $\tau_{\text{inc}}$  and  $\eta_{\alpha}$  corresponding to Fig. 7

	unit	NBI-co	NBI-counter	ICRH
$\tau_{\text{inc,tot}}$	ms	32	18	17
$\tau_{\text{inc,kin}}$	ms	18	12	12
$n_{e0} \Delta T_e / P_{\text{aux}}$	keV $10^{13} \text{ cm}^{-3} / \text{MW}$	1.25	0.78	0.95
$n_{e0} \Delta T_i / P_{\text{aux}}$	"	1.46		0.77

From Fig. 7a and Table 1 it appears that the confinement times for the total energy or its thermal part are larger for the best NBI-co conditions and are the same for the best NBI-counter and ICRH results. The ion contribution to the thermal energy  $E_{\text{kin}}$  is the largest for NBI-co and the smallest for ICRH.

On Fig. 7c we have compared the measured loop voltage values with those computed from neoclassical resistivity including bootstrap current. A good agreement is observed. The influence of ICRH on the sawteeth period has been discussed in Weynants *et al.*, 1989b.

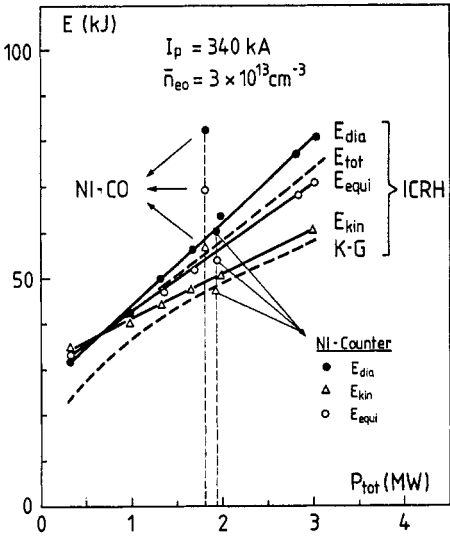


Fig. 7a Power scan of  $E_{\text{equi}}$ ,  $E_{\text{tot}}$ ,  $E_{\text{dia}}$ ,  $E_{\text{kin}}$  for ICRH. The representative points for NBI-co and NBI-counter heating conditions are given for comparison. The corresponding K-G scaling is also shown.

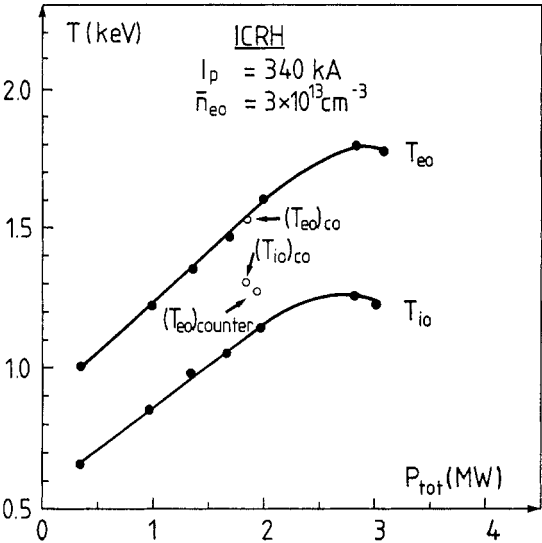


Fig. 7b  $T_{\text{eo}}$  and  $T_{\text{io}}$  evolution corresponding to Fig. 7a.

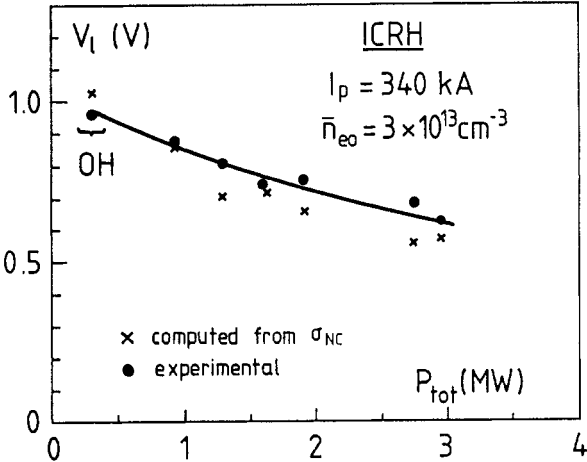


Fig. 7c

Power scan of  $V_l$  with ICRH. The crosses indicate the results of  $V_l$  computation (using  $\sigma_{\text{NC}}$  and taking into account  $I_{\text{BS}}$ ).

5. COMBINED HEATING SCHEMES

5.1 Combined NBI-co and ICRH

Figure 8 illustrates the evolution of plasma parameters during a NBI-co pulse to which an ICRH pulse of the same power is added. The effects induced by the addition of ICRH are striking :



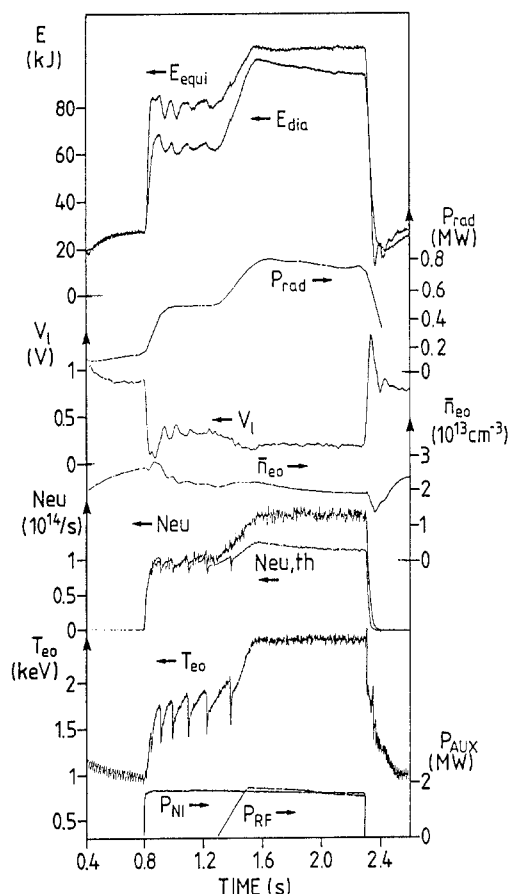


Fig. 8 Time evolution of  $E_{\text{equi}}$ ,  $E_{\text{dia}}$ ,  $P_{\text{rad}}$ ,  $V_l$ ,  $\bar{n}_{e0}$ , the neutron yield  $\text{Neu}$ ,  $T_{e0}$ ,  $P_{\text{NI}}$ ,  $P_{\text{RF}}$  during a NI-co followed by NBI-co + ICRH heated shot (# 41130).  $\text{Neu}_{\text{th}}$  gives the prediction of a theoretical model.

(i) It produces a more "isotropic" tail, i.e.  $E_{\perp} \cong 2E_{\parallel}$ , as seen from the evolution of the energies. A detailed analysis shows that  $E_{\parallel}$  remains unchanged within the error bar and that the effect of ICRH is to increase  $E_{\perp}$  roughly from  $E_{\parallel}/2$  to  $2E_{\parallel}$ . The effect on the total energy and on its thermal part is shown in Fig. 9a. The doubling of the tail energy content results from the formation by ICRH of its own minority energetic tail and from the enhancement of the energy slowing down time of the beam due to the hotter target plasma. The possibility of a further direct interaction between RF and beam particles will be discussed in Conrads *et al.*, 1990. The increase of the central electron temperature and of the ion temperature measured by charge exchange emission 19.5 cm off axis (i.e. at  $r/a = 0.25$  to 0.30 depending on the exact position of the magnetic axis) are shown in Fig. 9b and 9c. Large outward shifts of the electron and density profiles are observed caused by the high value of  $\beta_p$ . A peaking of both profiles with respect to the OH condition also occurs.

(ii) A further reduction of the loop voltage, which remains stationary during combined heating, is observed. The corresponding experimental points are shown in Fig. 3c. Such an additional drop is a result of a complex balance between the decreased resistivity and the increased beam slowing down due to the hotter target plasma, and a possible direct interaction of the RF and the beam at  $\omega = 2 \omega_{CD}$  resulting in increased slowing down as well as possible increased particle trapping. The increase of the total non inductively driven current  $I_{\text{ID}}$  can be obtained from Equ. (1). At  $\bar{n}_{e0} = 1.7 \times 10^{13} \text{ cm}^{-3}$ ,  $I_{\text{ID}}$  rises from 160 kA with 1.7 MW of NBI-co to 240 kA with the addition of 1.7 MW of ICRH. More than 2/3 of the plasma current (340 kA) is then non-ohmically driven; the contribution of the bootstrap current to the latter amounts to 45 kA (Conrads *et al.*, 1990). The changes of the target plasma parameters due to ICRH can only explain a 30 % increase of  $I_{\text{ID}}$ .

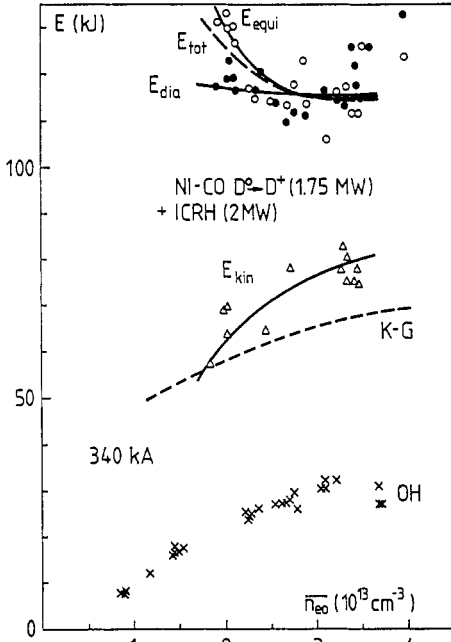


Fig. 9a Density scan of the energy for combined NBI-co + ICRH heating. The OH values and the K-G scaling predictions are also given.

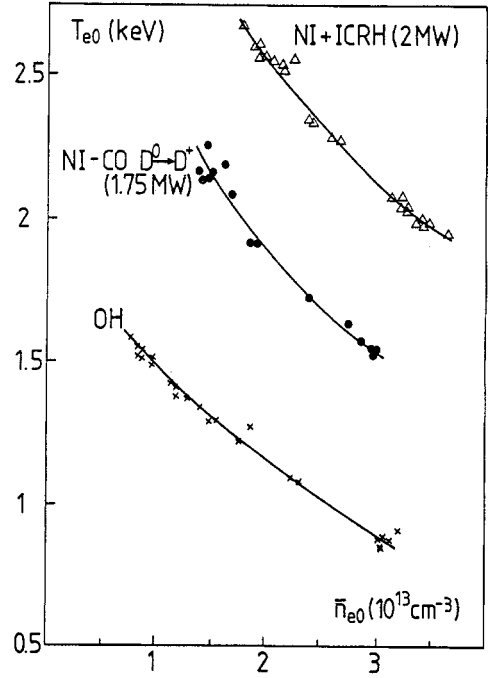


Fig. 9b Density scan of  $T_{e0}$  for OH, NBI-co and NBI-co + ICRH conditions.

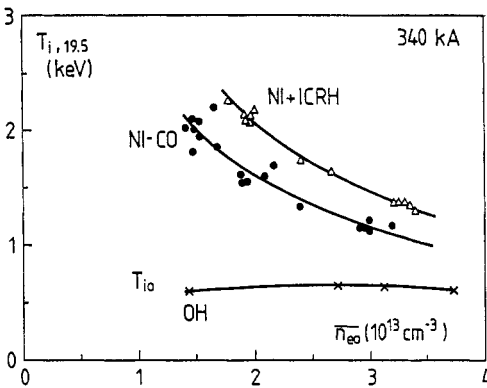


Fig. 9c Same as Fig. 9b but for  $T_{i0}$ .

(iii) In combination with NBI-co, ICRH is capable of extending the density domain in which full stabilization of the sawtooth occurs or in which a given stretching of its period is present. This effect can be interpreted by the large increase of the perpendicular tail energy which leads to an enhanced trapped hot-ions population inside the  $q = 1$  surface. In Fig. 8, it is seen that complete stabilization is not occurring with NBI-co at  $\bar{n}_{e0} = 2 \times 10^{13} \text{ cm}^{-3}$  but is occurring with the addition of ICRH. The rapid decrease of  $T_{e0}$  after the switch-off of both additional heating mechanisms is due to the crash of the monster sawtooth which occurs 10 ms later.

(iv) When the beam is present the neutron production is dominated by beam-plasma interaction. The application of ICRH with an equal amount of power as for NBI leads to an increase of the neutron yield

which may reach 80 %. This increase can be partly explained, as for the other effects, from the increase in the electron and ion temperature and possible changes in density of the target plasma but, in the best cases, a residual contribution remains unexplained. This is shown in Fig. 8 by the gap between the experimental curve  $N_{eu}$  and the curve  $N_{eu,th}$  which gives a theoretical prediction of the neutron yield. This prediction, which takes into account the beam and the target plasma characteristics, agrees well during the pure beam phase with the experiments but is unable to explain all the observed increases of the neutron yield during the combined heating (Van Wassenhove *et al.*, 1990). This means that an additional interaction mechanism to that of the modification of the target plasma has to be invoked, for instance a direct acceleration of the beam ions by the RF fields, to explain the total neutron production as well as the total beam driven current.

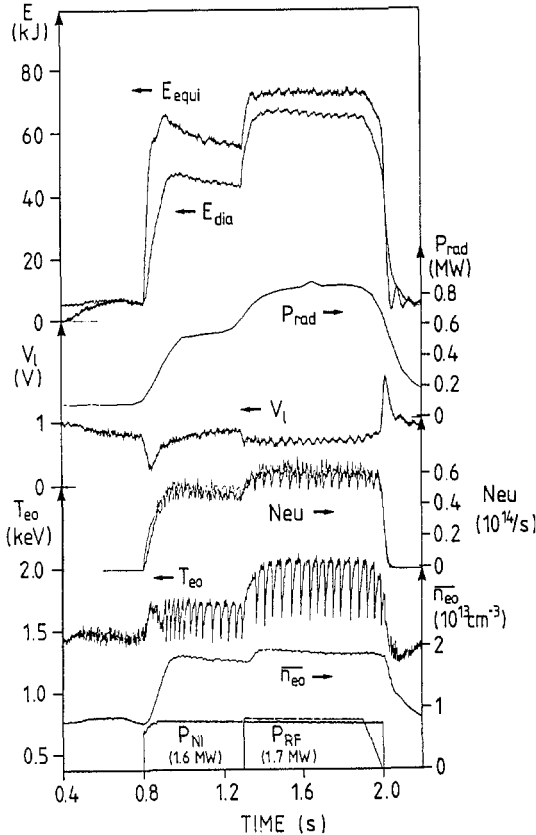


Fig. 10 Same as Fig. 8 but for NBI-counter and NBI-counter + ICRH (# 42092).

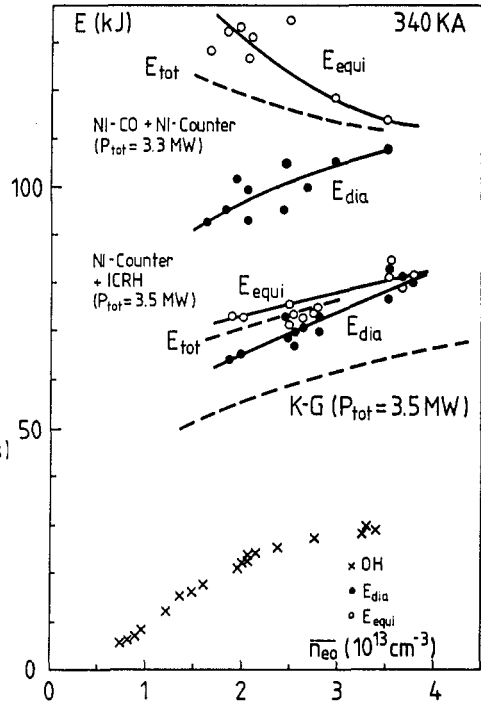


Fig. 11 Same as Fig. 9a but for NBI-counter + ICRH and NBI-counter + NBI-co.

## 5.2 Combined heating with counter-injection

### (i) NBI-counter combined with ICRH.

Figure 10 shows the time evolution of a shot heated by counter-injection to which an equal amount of ICRH power is added. As for the combination NBI-co + ICRH, a more "isotropic" tail is produced by the addition of ICRH and  $P_{rad}$  does not increase in proportion to the increased  $P_{tot}$ ; however for the same value of  $P_{tot}$  the energies and the central electron temperature are lower, and their increments with respect to the pure NBI heated conditions are also smaller. This appears also in the density scan of  $E_{dia}$ ,  $E_{equi}$ ,  $E_{tot}$  and  $T_{eo}$  given in figs. 11 and 12. Larger heating performances of NBI-co + ICRH with respect to NBI-counter + ICRH have also been observed in ASDEX (Ryter *et al.*, 1989). A peaking of the density profile, comparable to those in the case of NBI-co + ICRH, also occurs. The loop voltage drop induced by ICRH is small. This can be interpreted as a partial cancellation of two opposite effects : the electron temperature and bootstrap current rise, leading to a  $V_L$  drop and, as for

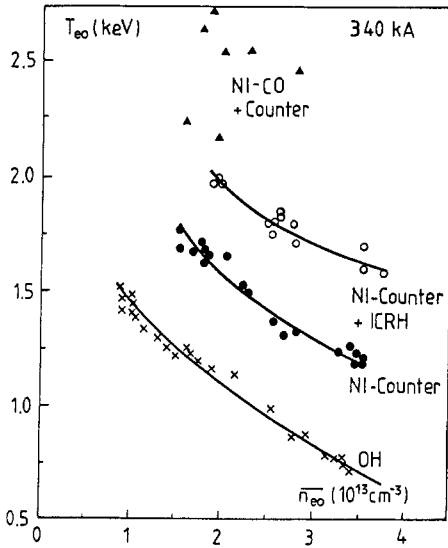
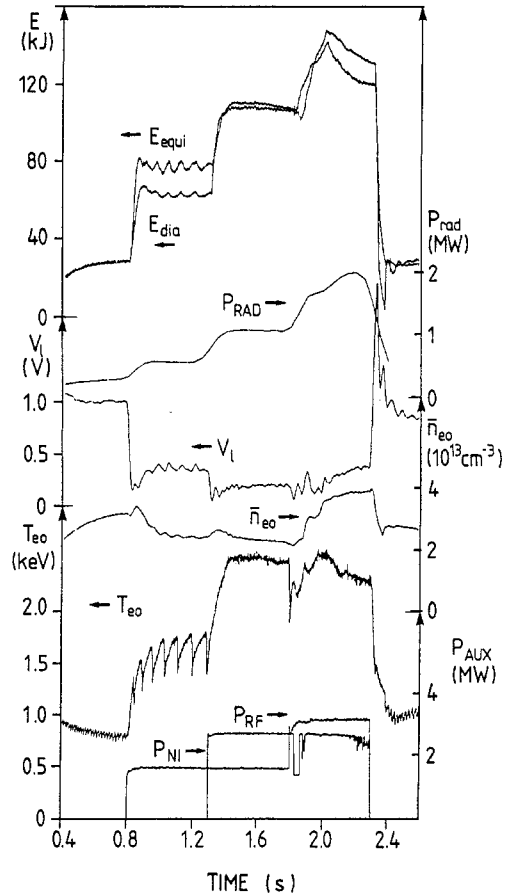


Fig. 12  $T_{eo}$  versus  $\bar{n}_{eo}$  for OH, NBI-counter, NBI-counter + ICRH and NBI-counter + NBI-co conditions. The conditions for combined heating are the same as in Fig. 11.

Fig. 13 Same as Fig. 8 but for NBI-co followed by NBI-co + ICRH, followed by NBI-co + ICRH + NBI-counter heating with a total auxiliary power up to 5.9 MW (# 41122).



the combination NBI-co + ICRH, an increase of the beam driven current opposite to the plasma current. The effect of ICRH on the sawtooth stretching is much smaller than for its combination with NBI-co. Comparing Fig. 10 and Fig. 8, it is seen that the neutron yield (noted Neu) is smaller for NBI-counter heating but that it is also increased by the addition of ICRH.

(ii) NBI-counter combined with NBI-co.

A large scatter of the results is observed. It results from non stationary conditions due to a density rise during the heating pulse, sometimes accompanied by an impurity concentration increase. In Figs. 11 and 12 the best energy and  $T_{eo}$  results achieved up to now for balanced injection are shown. They are characterized by large energy anisotropy and a much higher value of  $E_{tot}$  than for the combination NBI-counter + ICRH for the same  $P_{tot}$ . The value of  $T_{eo}$  is also higher. The density profile becomes highly peaked, more than for the other combinations. The loop voltage roughly remains at the level reached with pure NBI-co. This indicates a compensation between the effect of the electron heating and of the bootstrap current increase and the drop in the total beam driven current. Complete sawtooth stabilization can also be achieved. A detailed study of its parametric dependence and its link with the theoretical explanation (Ongena *et al.*, 1990) is still to be done.

(iii) All heating methods combined.

The addition of NBI-counter to a shot already heated by NBI-co + ICRH leads to the largest energy content and  $\beta_p$  (~1.5) reached in TEXTOR. An example is shown in Fig. 13. The characteristics are the same as for the combination NBI-counter + NBI-co, except that the energy tail becomes more

"isotropic" ( $E_{\perp} \approx 2E_{\parallel}$ ). Note the sawtooth stabilization which remains after an initial crash at the start of NBI-counter, the large  $\pi_{e0}$  rise which explains the lack of  $T_{e0}$  increase, the  $V_{\ell}$  behaviour which remains initially to its level with NBI-co + ICRH. During the NBI-counter pulse an increase of the oxygen concentration is observed : this is to be correlated with the increase of  $P_{rad}$ , the decrease of  $E_{dia}$  and  $E_{equi}$  and the rise of  $V_{\ell}$  which occur for  $t > 2$  s.

(iv) Comparison between the combinations of two out of three methods.

In Table 2 a comparison is given between the best results of  $\tau_{inc,tot}$ ,  $\tau_{inc,kin}$  and  $\eta_e$  achieved for  $\pi_{e0} = 3 \times 10^{13} \text{ cm}^{-3}$  and  $I_p = 340 \text{ kA}$  for the different combinations (with  $P_{NBI-co} \equiv P_{NBI-counter} \equiv P_{RF}$ ).  $\tau_{inc,kin}$  is not available for NBI-counter + ICRH due to the lack of  $T_i$  measurement.

Table 2 :  $\tau_{inc}$  and  $\eta_e$  comparison ( $\pi_{e0} = 3 \times 10^{13} \text{ cm}^{-3}$ ,  $I_p = 340 \text{ kA}$ )

	unit	NBI-co + ICRH	NBI-count. + ICRH	NBI-count. + NBI-co
$\tau_{inc,tot}$	ms	24	14	28
$\tau_{inc,kin}$	ms	13	-	(21)
$\eta_e$	keV $10^{13} \text{ cm}^{-3}/\text{MW}$	1.0	0.75	1.5

For the combination NBI-co + NBI-counter, we use results of shot simulations by TRANSP yielding  $E_{\parallel} \sim 1.6 E_{\perp}$  to evaluate  $E_{kin}$  from  $E_{dia}$  and  $E_{equi}$ . From Table 2, it results that the combinations NBI-co + ICRH and NBI-counter + NBI-co are roughly as effective for the total energy,  $T_{e0}$  increases and are more efficient than the combination NBI-counter + ICRH.

## 6. CONCLUSIONS

Different auxiliary heating methods and their combinations have been applied to and compared on TEXTOR. A large energy anisotropy is observed in all cases and has been studied in terms of the thermal and the tail part of the energy content. Large enhancement with respect to the Kaye-Goldston scaling is found as well with NBI-co or the combinations NBI-co + ICRH or NBI-co + NBI-counter even when the engineering value of  $P_{tot}$  is taken to compute the K-G scaling. For the total energy

content  $E_{tot}$  the enhancement can reach about 2.3 for the latter two combinations at  $\pi_{e0} = 1.75 \times 10^{13} \text{ cm}^{-3}$ . For the thermal energy  $E_{kin}$  the largest deviation is obtained for densities larger than  $3 \times 10^{13} \text{ cm}^{-3}$  and reaches then 1.5 for the same heating combinations. With the three heating methods combined, a value of  $\beta_p = 1.5$  has been reached corresponding to a  $\beta_t = 0.65 \%$  which is equal to 70 % of the Troyon limit of  $2.8 I_p (\text{MA}) / (a (\text{m}) \times B_t (\text{T}))$ . It has been shown that complete sawtooth stabilization of the "monster" type can be obtained by the hot ion tail produced by NBI-co and that the density domain in which stabilization occurs can be significantly extended by the addition of ICRH (e.g. from  $n_{e0} < 1.5 \times 10^{13} \text{ cm}^{-3}$  up to  $n_{e0} < 2.25 \times 10^{13} \text{ cm}^{-3}$  by the addition of 1.4 MW of ICRH). Sawtooth stabilization is also observed for balanced injection. High non inductively driven current is observed with NBI-co which is increased by 50 % by the addition of ICRH, allowing to

drive up to two third of the plasma current at low density ( $\pi_{e0} = 1.7 \times 10^{13} \text{ cm}^{-3}$ ). The effect of counter beam driven current and of its increase in presence of ICRH is clearly seen with NBI - counter injection.

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