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**A Monte Carlo Program System for
Beam-Materials Interaction Studies**

by

P. Cloth, D. Filges, R.D. Neef, G. Sterzenbach,
Ch. Reul, T.W. Armstrong, B.L. Colborn,
B. Anders, H. Brückmann

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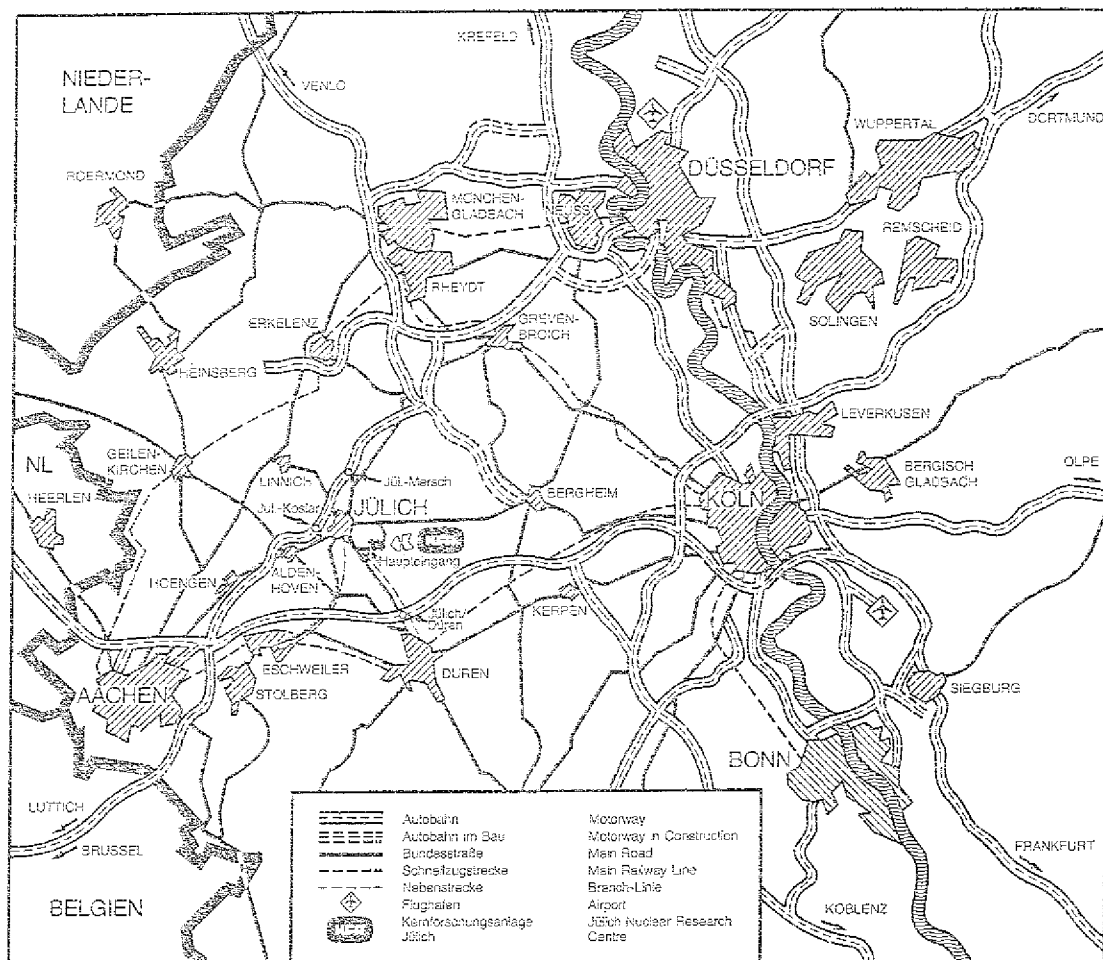
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Hanno Brückmann †

With deep sorrow we remember the sad demise of one of our co-authors, Professor Dr. Hanno Brückmann, on 11th November 1987 during the work on this report.

Hanno Brückmann was the dynamic inspiration behind the creation of the HERMES system. We will miss him personally and his valuable professional guidance.

The Authors

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1. INTRODUCTION

1.1 Overview of the HERMES System

HERMES (High Energy Radiation Monte Carlo Elaborate System) is a system of Monte-Carlo computer codes that are necessary to treat the different physics to be considered in computer simulation of radiation transport and interaction problems. In addition HERMES offers powerful capabilities of Monte-Carlo analysis and of merging the outcomes of partial programs into final results. An interfacing system - the "HERMES submission file" - provides easy interchange of input/output data between programs by standardized data structures and allows unproblematic extension of the HERMES system by introducing new programs.

The HERMES collection of physics programs permits the simulation of secondary particle histories induced by primary particles of any energy up to the regime of high-energy physics and down to thermal energies, e.g. for neutrons. The particles, that are considered by the programs of the HERMES system are p , n , π^+ , π^- , π^0 , $\mu^\pm$, e^+ , e^- , γ , and light ions to $A=10$. The development of particle cascades can be simulated within very complex geometries and material configurations with only minor restrictions.

The geometry description is identical for all Monte-Carlo programs incorporated in HERMES and made in input terms of the approved "Combinatorial Geometry (CG)" in a KFA extended version.

The well established programs of the HERMES system in general have been taken as original codes as far as possible. However, to satisfy the needs of some applications, e.g., the high energy calorimeter simulation, some extensions and changes became necessary. Also the interfacing technique by HERMES submission files needs some additional programming. Therefore, the name of the original codes were extended by a KFA suffix. All changes made to the original codes are documented within this report. For simplicity reasons, however, we refer to the original names throughout this document, e.g. HETC rather than HETC-KFA 2. Several of the system codes, especially for analysis purposes, are provided by the authors.

To achieve portability of the HERMES system, the original coding of the programs was converted to the FORTRAN77 standard. This, however, was verified only on two computer systems, the IBM 3081 and the CRAY-XMP22.

1.2 Physics with HERMES

The physical phenomenon to be simulated with HERMES is the history of the secondary particles induced in matter by an incident high energy particle. This typical propagation process of high energy radiation in matter in general is referred to as a particle cascade.

To demonstrate the range of physics covered by HERMES we give a brief overview of the basic phenomenology associated with the interaction of high energy particles with matter and with the transport of such high energy radiation, as it may be produced in particle accelerators or exists as cosmic radiation in space.

The energy losses of high energy particles (> 1 GeV) travelling through matter are mainly determined by production of secondary particles. Thus, the main feature of the cascade is an initial increase of the particle intensity with depth and time. If the energy of the produced secondary particles is high enough, they in turn produce additional particles. There exists however a physical limit for the development of the cascade, because the initial energy of the primary particle is dissipated all over the produced particles. Therefore, the multiplicities tend to decrease again during the cascade process and fade away because the average energy of the cascade particles decreases and a greater fraction of the individual particle energy is now dissipated by ionization losses. At the end of the cascade process subsequent production of many low energy particles, mainly neutrons, takes place, known as the evaporation process.

The physics of a high energy cascade in matter is to some extent determined by the type of the incident primary particle. If we are bombarding matter with high-energy hadrons (protons, neutrons, pions etc.) a so called hadronic cascade is produced, which is often accompanied by an electromagnetic cascade depending on the energy of the primary particle. If the primary particle is an electron, a positron or a photon, a purely electromagnetic cascade is produced.

The interaction process of particles with the nuclei of matter may be considered as a particle cascade inside the nucleus, thus the term "intranuclear cascade" is used to describe the process creating a particle cascade inside the nucleus. The nuclear collision products can proceed to produce further collisions with other nuclei to form the "internuclear" cascade.

To analyze the particle shower and its distribution in energy and space, we have to determine the multiplicities, energy and angular distributions of the particles generated by suitable simulation programs. The development of electromagnetic showers with their principal production processes - bremsstrahlung for electrons and positrons, pair production for photons, becoming energy dependent above 1 GeV - are very well described by quantum electro dynamics (QED-theory) over a wide energy range. In HERMES this is taken into account by the EGS4 code.

In contrast to the electromagnetic shower, physics is considerably more complex for hadronic showers, therefore, some restrictions exist on determination of the shower parameters in detail as a function of energy in consistent theory, e.g. quantum chromo dynamics (QCD theory). To study the relevant physics associated with the interaction of high-energy hadrons with matter, a very potent theoretical model, the "Intra-Nuclear-Cascade-Evaporation" model (INCE), treating nonelastic and elastic interaction of high-energy hadrons with nuclei, is available in the HETC (High Energy Transport Code) Monte Carlo program.

Full use of the above described cascade physics, which can be investigated by HERMES simulation, is made in high energy physics, especially considering high-energy calorimeter techniques. All parts of the cascade contribute significantly to the responses of these detectors frequently used in high energy physics. To get an impression of the HERMES capabilities, one may consider the sample problem (a) in Appendix E.

The structure of the HERMES system is also highly in favour of simpler or partial problems as compared to the calorimeter physics, e.g., problems which can be solved with only one of the physics programs. In addition the system allows further treatment of all those problems for which the single physics programs have been originally designed. However, with partly considerable extended and enhanced analysis features. Two examples for this type of application - namely cross-section computation with HETC and thick-target particle flux calculations using HETC and MORSE-CG are also given in Appendix E.

2. GENERAL DESIGN CHARACTERISTICS OF THE HERMES MONTE-CARLO SYSTEM

To simulate the histories, i.e. production, interaction with matter and transport, of high energetic particles the latest state-of-the-art radiation transport codes and event generators are used in HERMES. All the codes employ Monte Carlo techniques with 3 dimensional geometry description, therefore, it is possible to treat in detail the reaction mechanism and the transport of the high energy particles as well as of the low-energy particles created. An overview of the main physics models is given in the following section with some detailed description of the codes HETC, MORSE, and EGS. All codes make use of a common data interface named "HERMES Submission File" (see Section 2.5).

2.1 Scheme of HERMES Performance

The problems which can be solved with the aid of the HERMES system cover a wide range of incident particle energies and a set of particle types. Therefore, the computational work had to be shared between several Monte-Carlo codes, each solving a specific part of the general problem. Currently three Monte-Carlo codes are implemented, namely HETC, MORSE-CG and EGS4.

We decided to use offline coupling of the codes, due to practical and scientific reasons:

- With off-line coupling it is much easier to implement additional Monte Carlo codes or to replace codes. Thus, HERMES can be extended in an easy way to represent the state of the art, without need for changing of programs.
- Some problems, that require a large amount of computer power (storage and CPU time), can be partitioned in steps.
- The results of each step can be examined to find out errors as well as the statistical and physical quality of intermediate results. Subsequent steps can be tailored according to the statistical properties of prior steps.
- Intermediate results may also improve the understanding of the problem of the physical models used and the dependency between these models.

The coupling procedures affect the flow of particles through the system and the collection of results. We designed a general data structure used by the programs in HERMES,

- to accept particle sources generated in other programs,
- to submit particles for further treatment in other programs,
- to submit detector results at the end of each independent batch and at the end of a run.

A detailed description of the communication file structure is given in Section 2.5. Particle and detector data flow is illustrated in Fig. 2.1.

Organization of HERMES

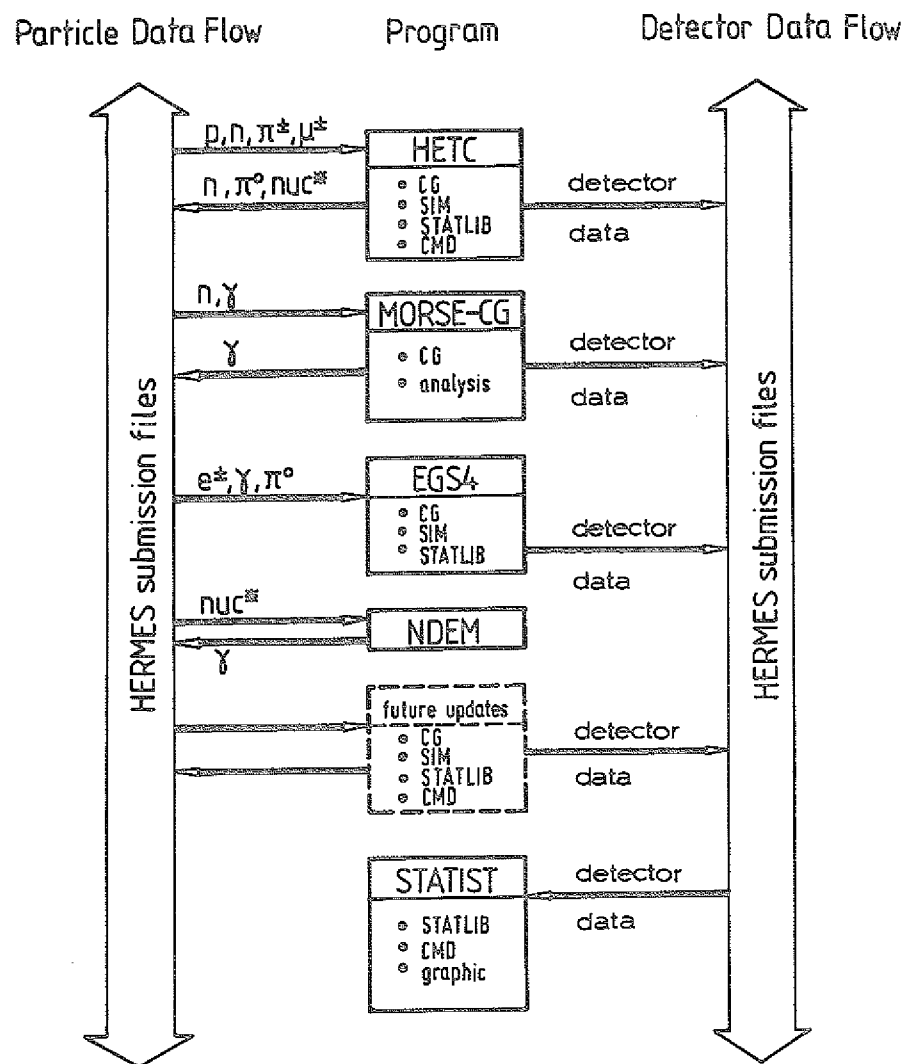


Fig. 2.1: Organization of the HERMES (High Energy Radiation Monte Carlo Elaborate System) program system for theoretical studies on beam materials interaction

Whenever a particle cannot be handled by a program, it may be submitted to a HERMES submission file. Also the residual nuclei from HETC are submitted to that file. Particles can be picked up directly by another Monte-Carlo program. The residual nuclei, however, are processed by the auxiliary code NDEM (see Section 2.2.4) which computes the de-excitation γ 's from these nuclei and submits the γ 's to another submission file.

In some cases (if no strict correlation is needed) it may be sufficient to pick up only a representative subgroup of all the particles found on a submission file. (In a later release of HERMES) we will provide a code to select particles from submission files, merge submission files and examine their contents. The detector data submitted by the Monte-Carlo codes working on the same problem can be handled by the auxiliary code STATIST (see Section 2.3.3).

We provide some subroutine packages with HERMES to aid the Monte Carlo codes in solving problems, which are the same for each of the Monte Carlo codes.

The most important subroutine package is the Combinatorial Geometry CG-L2 (see Section 2.3.1), which we have implemented in HETC, MORSE-CG and EGS4. Therefore, the geometry description of a problem can be made once for all codes. The Combinatorial Geometry has been increased in its capabilities. A test program has been developed so that extensive testing of the geometry setup can be made prior to the Monte Carlo runs. Additionally we provide a set of routines named STATLIB which are used in the analysis part of all programs and an input interpreter CMD (see Section 2.3.4), which provides a standard tool to express input for the Monte-Carlo codes. In the current release we use CMD to setup the geometry input, test the geometry, setup the input for the HETC-analysis code SIM and for the final statistics code STATIST, which is used to combine the results of different Monte-Carlo codes working on the same problem.

In the analysis of coupled Monte-Carlo runs we use trigger events to identify the results coming from the same incident sample. This sample may consist of one or more source events. In applications, where mean values are of interest it is suitable to use batches of more than one source event, whereas in

applications, where the fluctuations of physical properties are also of main interest, it is necessary to use one source event per batch. The statistical analysis is done on the basis of samples (i.e. batches).

It should be noticed, that the measurement of fluctuations prohibits the use of most of the biasing methods, which are commonly used in Monte-Carlo programs to reduce the computation time and increase the stability of mean value estimates. Therefore, the Monte Carlo codes used in HERMES have been modified to allow a maximum of analog (i.e. also unbiased) performance.

2.2 The Physics Modules

2.2.1 The Hadron Cascade Module HETC-KFA-2

HETC (High Energy Transport Code) is implemented in a version based on the original coding given in Ref. /1/. An overview about changes, updates and physics involved is given below (for details see Appendix A). The user, who is not familiar with the original HETC should also read Refs. /1,2,3/.

Name:	HETC-KFA-2	(High-Energy-Radiation-Transport-Code, KFA Version 2) For simplicity reasons we refer to this version also as HETC in this document.
Implementation:	based on Ref. /1/ (Report ORNL-4744)	
Extension:	FORTRAN77 compatible on IBM3081 and CRAY-XMP22	
Particleas transported:	$n, p, \pi^{\pm}, \mu^{\pm}$	
Extension:	d, t, He^3, α , heavy ions $A < 20$	
Mechanism:	- collision model - Intra-Nuclear-Cascade-Evaporation (INCE) (for $p, n, \pi^{\pm}; \pi^{-}$ capture at rest; non-elastic nuclear collisions; $\pi^0, d, t, He^3, \alpha$-production; residual nuclei and residual excitation energy) - charged particle ionization energy loss (dE/dx) - primary particle small angle multiple	

Coulomb scattering

- decay (weighted fashion) for $\pi^\pm$ and $\mu^\pm$ in flight and at rest
- scaling model - automatic parameter update for nonelastic collisions above 3.5 GeV
(limited energy range for scaling
p/n - $E_{\max} \leq 25$ GeV
 $\pi^\pm$ - $E_{\max} \leq 15$ GeV)

Extensions:

- elastic collision module for p/n elastic nucleus collisions up to 20 GeV
- high energy fission included in evaporation model
- automatic parameter update for B_0 in level density formula
- nonisotropic angular distribution of evaporated particle
- analog decay for $\pi^\pm$

Geometry:

standard CG geometry interface

Extension:

CG-KFA geometry module (see Section 2.3.1)

Additional

submission of particles and nuclei for

Extensions:

further processing (see Section 2.5)

structure of HERMES submission files)

2.2.2 Neutron- γ Transport Code MORSE-CG-KFA

MORSE-CG (Multigroup Oak Ridge Stochastic Experiment with Combinatorial Geometry) was implemented in a KFA version based on the original code (see Appendix A).

The user, who is not familiar with MORSE should read Refs. /4,5,6/. MORSE does a stochastic simulation on neutral particle transport.

Name: MORSE-CG-KFA (Multigroup Oak Ridge Stochastic Experiment)

In this document we refer to this version also as MORSE-CG.

Implementation:

based on Ref. /4/ (Report ORNL-4972)

Extension:

FORTTRAN77 compatible on IBM3081 and
CRAY-XMP 22

- Source:** fixed neutron- or γ -source or fission feedback
- Extension:** submitted neutron- or γ -source distribution in phase space (see Section 2.5 about the structure of HERMES submission files)
- Mechanism:**
- Elastic neutron nucleus collisions including hydrogen collisions; non-elastic neutron nucleus collisions; inelastic neutron nucleus collisions; capture and fission; γ production and transport (capture, fission, inelastic, and nonelastic)
 - analysis of fluxes and reaction rates in phase space
 - variance reduction methods: russian roulette, particle splitting, path length stretching, simulated absorption, and productive nuclear collision (n,2n, etc.), importance sampling, leakage rejection and albedo scattering
- Extensions:**
- modifications in some parts from weighted to analog Monte Carlo (particle absorption using russian roulette procedure, fission, γ -generation, inelastic scattering using splitting feature) to allow analog treatment of particle yields for calorimeter design
 - simultaneous implementation of volume-, surface-, and point-detectors
 - escape, leakage current and fluxes at surfaces
 - features in calculating light response functions for scintillators
- Geometry:** standard CG geometry interface
- Extension:** CG-KFA geometry module (see Section 2.3.1)
- Additional extensions:**
- submission of fluxes and reaction rates for further processing (see Section 2.5) (e.g. light response or visible energy in scintillators)
 - binary storage of results on RSYST /7/ data base
 - revision of input to allow fast vector assignment using REA formats
 - γ rays are stored via HERMES submission files for EGS transport

Cross section standard:	- Multigroup cross section libraries for neutron and γ 's or coupled neutron- γ libraries in ANISN or DTF4 format
Extension	- multigroup cross-section libraries in RSYST format

2.2.3 The Electron-Photon Transport Module EGS4-KFA

The EGS4 (Electron Gamma Shower) was implemented in a KFA version based on the original code (see Appendix A). The EGS computer codes provide Monte-Carlo simulation of the coupled transport of electrons and photons with energies from a few keV upto several TeV. The user, who is not familiar with the EGS4 code should read Refs. /8,9/.

Name:	EGS4	(Electron Gamma Shower)
Implementation:	based on Ref. /9/ (Report SLAC-265)	
Extension:	Standard user code (see Section 3.4) FORTRAN77 compatible on IBM3081 and and CRAY-XMP 22	
Source:	e^+ , e^- , γ 's	
Extension:	submitted electro-magnetic particle sources (e^+ , e^- , γ 's in phase space) (see Section 2.5 about structure of HERMES submission files)	
Mechanism:	- small and wide angle scattering - bremsstrahlung, pair production, Compton scattering, Moeller effect, Bhabha effect, photo-electric effect - annihilation - continuous e^+, e^- energy loss - step length correction (Rogers formalism)	
Additional Extensions:	- submission of electron-, positron- and gamma fluxes and currents, energy depo- sition and reaction rates (see Section 2.5 about structure of HERMES submission files)	

2.2.4 The Nucleus de-Excitation Module NDEM

The NDEM code is used to generate a γ -ray source from the de-excitation of residual nuclei, the assumption is made that all particle decay modes have been exhausted; thus γ emission does

not compete with particle emission. The NDEM formalism is taken from the Los Alamos PHT code /10/. The input to NDEM is a HERMES submission file of the residual nuclei information recorded by HETC. The γ ray source also on a HERMES submission file can be input into MORSE-CG or EGS4.

It is assumed that there exists a range of known energy levels given by the Gilbert Cameron formulae /11/. The library of level data is based on the CDRL82 library with over 50,000 listed levels. If the excitation energy of a residual nucleus lies between two levels, it is assigned randomly to the one or the other of the levels with a probabilities that depend linearly on the distances from these levels. If the excitation is above the maximum known level, the level is assumed to be in the continuum.

The probability of a transition between levels is assumed to be proportional to the Weisskopf single-particle estimates /12/, unless specified by the library. For levels in the continuum and for known levels with unknown spin and parity, a spin state is randomly assigned using the Gilbert Cameron spin densities to generate a sampling distribution; parities, at the present time, are assigned with equal probabilities. For a transition within the continuum to a state of given spin, the transition probability is also proportional to the level density for the final spin state.

The gamma cascade proceeds with the above assumptions. For an excitation energy lying in the continuum, transition to another level in continuum competes with transitions to known levels. If the initial excitation energy lies within the known levels, or a level has been reached by transition from the continuum, the probability of a transition to lower levels is obtained from library data, if available, or from the model otherwise. When specified in the library data, a 0^+ to 0^+ transition is permitted to occur without γ emission (representing internal conversion).

The user has the option of selecting the level of the physics model to be used in the NDEM calculation. It is possible to ignore the experimental branching ratios and perform a pure model calculation. The user may also choose to treat all transitions as E1 transitions, corresponding to the model previously employed by Troubetzkoy /13/.

At the present time, the emission of de-excitation γ 's is assumed to be isotropic in the lab frame, ignoring nuclear recoil.

2.3 Geometry and General Analysis Programs, Statistics and Auxiliary Codes

2.3.1 Three Dimensional Geometry Model based on Combinatorial Geometry

(1) Conception of the Combinatorial Geometry

The Combinatorial Geometry cuts space into pieces called input-zones. Input-zones consist of one or more code-zones. Code-zones are composed of one or more bodies. A problem description as input to the Combinatorial Geometry starts with the definition of the bodies:

Definition of Bodies

Bodies are defined by the body type (sphere, box, etc.), an identification number and parameters which define location and dimensions of the body.

For example a sphere with midpoint coordinates (1,2,2) and radius 6 can be described as follows:

SPH	8	1.0	2.0	2.0	6.0
-----	---	-----	-----	-----	-----

Where "SPH" identifies the body as of type sphere; the reference number is 8.

Each body divides space into two pieces: The area inside the body and the area outside the body.

The area inside the body can be referenced by a positive body identification number, whereas the area outside the body is referenced by a negative body identification number.

Definition of Code-Zones

Code-zones are coded as a sequence of body inside/outside references, e.g.,

+3	+4	-1
----	----	----

is a code-zone consisting of all points lying within body 3, within body 4, but outside body 1.

Note, that code-zones are intersections of body inner and outer areas!

In the example above points lying within body 3, but not within body 4 do not belong to the specified code-zone.

Note, that all space must be completely defined by code-zones and that code-zones should not overlap!

Definition of Input-Zones

Input-zones are coded as a sequence of code-zones combined by the OR operator. For each input-zone a medium and region number is specified, e.g.,

```
1 100      +3      +4      -1OR  +3      -4
```

defines an input-zone with medium number 1 and region number 100. It consist of all points inside body 3 and inside body 4 but outside body 1 and all points inside body 3 and outside body 4.

Note, that input zones must not overlap!

When all zones are defined, every point in space will belong uniquely to a medium and a region number.

Some medium numbers are reserved for special purposes:

- Medium -1 denotes the external void.
whenever medium -1 is entered , the transport of the particle is terminated.
Medium -1 should be used as the outer bound of a geometry. It is the only medium which may consist of infinite areas.
- Medium 1000 denotes internal void.
Internal voids can be used in MORSE to specify a vacuum area. Both the meaning of the calling parameters and the action of the Combinatorial Geometry program is different compared to normal media, when medium 1000 is encountered. For HETC and EGS4 one should use medium 6666, which is treated as any other medium in the geometry routines.
- Medium 999 is used to terminate zone-input.
No more information is required on that card.

For detailed information on input formats, body definitions, and extensions of this basic concept, refer to Section 3.9.

(2) KFA-Extensions of the Combinatorial Geometry

Two types of extensions have been incorporated into CG:

- additional bodies have been implemented, which help to avoid redundant surfaces, and numerical problems caused by multiply defined surfaces.
- a method is provided to divide zones into sub-zones.

(a) Additional Body Types

In basic CG only finite bodies like SPH, RPP or RCC are available. The use of these bodies can introduce unwanted or redundant surfaces and (even worse) multiply defined surfaces. Consider the following example:

To define a cylinder which is partitioned along the axis, one could use a series of RCC bodies.

```

RCC  1      0.      0.      0.      0.      0.      1.
      10.
RCC  2      0.      0.      1.      0.      0.      1.
      10.
RCC  3      0.      0.      2.      0.      0.      1.
      10.
END
  1  1  +1
  1  2  +2
  1  3  +3
999

```

Here we have multiply defined surfaces at $z=1$, $z=2$ and $r=10$. CG may have to calculate the same intersection point up to 3 times, possibly with slightly different results due to numerical uncertainties. This may cause numerical problems in the program, at least computational work is increased.

```

RCC  1      0.      0.      0.      0.      0.      3.
      10.
RCC  2      0.      0.      1.      0.      0.      15.
      15.
RCC  3      0.      0.      2.      0.      0.      15.
      15.
END
  1  1  +1  -2
  1  2  +1  +2  -3
  1  3  +1  +3
999

```

Above the same geometry is described without multiply defined surfaces, but this time 4 redundant surfaces are defined.

Such problems can be solved by defining semi-infinite bodies. We have implemented PLANES, where the half space lying in the normal direction of the plane is considered to be the inside of the body. The available planes are listed in Table 2.1. We also made available CYLINDERS of infinite length (Table 2.2).

Table 2.1
Infinite Planes as CG Bodies

body	type	parameter	points inside
PX	K	X	$x > X$
PY	K	Y	$y > Y$
PZ	K	Z	$z > Z$
P	K	UX UY UZ D	all points lying beyond a plane with normal direction (UX,UY,UZ) and distance D from origin.
PS	K	SX SY SZ UX UY UZ D	all points lying beyond a plane with normal direction (UX,UY,UZ) and distance D from (SX,SY,SZ)

The cylinders are specified by a point (SX,SY,SZ) lying on the axis, a direction vector (UX,UY,UZ) for the axis and the radius R.

Table 2.2
Infinite Cylinders as CG Bodies

body	type	parameter	axis direction	radius
CX	K	SX SY SZ R	(1,0,0)	R
CY	K	SX SY SZ R	(0,1,0)	R
CZ	K	SX SY SZ R	(0,0,1)	R
C	K	SX SY SZ UX UY UZ R	(UX,UY,UZ)	R

Using these bodies we can describe the above geometry example as:

```

CZ      1      0.      0.      0.      10.
PZ      2      0.
PZ      3      1.
PZ      4      2.
PZ      5      3.
END
  1      1      +1      +2      -3
  1      2      +1      +3      -4
  1      3      +1      +4      -5
999

```

(b) Subdivision of Zones

For many applications it is necessary to define mesh-like geometry structures. In basic Combinatorial Geometry such structures require large input (because each mesh element has to be defined separately), and much computer time is consumed because the CG algorithm cannot take advantage of the regular structure of a mesh. Normally this problem is solved by writing special problem dependent geometry codes. But this is also not convenient because of the large amount of manpower absorbed by such a work; furthermore it is not trivial to construct a geometry code, when the mesh structures are embedded in a non-regular environment.

Thus, a method has been developed to enable CG-L2 to handle hierarchically organized geometry structures. Additionally an input preprocessor (CMD) has been designed to minimize the amount of input data needed.

In CG-L2 the user may design the geometry in up to three levels of refinement. First the global zone structure is defined as usual. Each zone which shall be subdivided in a subsequent step is marked by the GO operator and the sequence number of the code zone card sets where the subzones will be defined. Additionally the region number is chosen as a negative integer number between -1 and -999. The absolute value of this number will be used as the first three digits of all subzone-region numbers associated with this zone. Also the medium number can be specified as an integer value less than -1 indicating the medium will be assigned in the definition of the subzones. The global zone structure is terminated by a card with IMED=777 (or SUBGEO; in CMD language). Then the second zone card-set starts, i.e., the card-set which is called with GO 3. Up to 10

card-sets may be specified. The last one must be terminated with 999 (or ENDGEO; in CMD language) as usual. The nesting level of GO operations is limited to three as mentioned above. Thus, the resulting region numbers have 9 digits (3 from each nesting level). The assignment of the medium number may occur in any of the 3 levels: A medium number > -2 in level one will be used for all subzones of level 2 and 3. A medium number > -2 in level two will overwrite the media assigned in level 3. Similarly the region number assignment is overwritten by the first positive region number found in a refinement path.

These conventions allow the use of subzones with the same medium but different region numbers or the same region but different medium numbers.

As an example we may consider a geometrical setup of a radiation shield. Fluxes shall be mapped with respect to the z-coordinate and the radius around the z-axis. First the shield is described as constructed. All media are assigned positive. The region numbers are set negative. A second zone card-set is used to define the z-structure of the flux mesh and a third to define the r-structure. All regions of level one, which are to be mapped have the GO 2 operator in the definition card. All z-zones defined in card-set 2 refer to the r-structure (GO 3). Then the regions found by the geometry will be of the form $[ireg_1] * 1000000 + [IZreg] * 1000 + [IRreg]$

2.3.2 The General-Purpose Analysis Module SIM

We have implemented an online-analysis subroutine package into HETC, which is used to provide scoring of the physical results of the HETC run. This package, named SIM, includes routines to read-in user analysis requests (coded as CMD-commands), to compute the contributions of an event to the detector signals, to provide estimates of the statistical errors of detector signals and to interface HETC to the final processing code STATIST. Table 2.3 presents the currently available detector types.

Table 2.3
Available SIM Detectors

Type	Symbol	Description
1	$Y_C(E,A,IR1)$	Yield of secondary particles from the intranuclear cascade (INC)
2	$Y_e(E,A,IR1)$	Yield of secondary particles from evaporation and fission model
3	$Y(E,A,IR1)$	Yield of secondary particles
4	$E(ctyp,IR1)$	Energy deposited by various deposition schemes
5	$L_s(ctyp,IR1)$	Scintillation light generated by various schemes
6	$J^+(E,A,IR1-IR2)$	Forward particle current through given surfaces
7	$\phi^+(E,A,IR1-IR2)$	Forward particle flux through given surfaces
8	$\phi(E,A,IR1)$	Particle tracklength flux in given regions
9	$E^+(ptyp,IR1-IR2)$	Particle energy passing given surfaces.

Functional Description of the SIM Analysis

SIM is called by HETC after each collision and after each particle flight. It scans all detectors defined to decide whether the event will contribute to the detector signal.

- First the particle type is checked against the required type in the DEFDET command.
- Then the region or surface index is found.
- Energy binning is performed, if requested.
 - when secondary particle yield, surface flux, current or neutron tracklength flux is measured, the actual energy of the particle is used to find the energy bin index.
 - when charged particle tracklength flux is measured, the flight path of the particle is subdivided into pieces which correspond to the path length travelled in the appropriate groups using the slowing down algorithm of HETC.

- Angular binning is performed, if required.
- If a detector reference direction is given in the DEFDET command, the angle is taken between the flight direction of the particle and a detector reference direction.
- If a detector reference direction is not given and yields are measured (detector type 1-3), then the angle is taken between the flight direction of the particle and the flight direction of the incident particle.
- If a detector reference direction is not given and surface flux or current is measured, then the angle is taken between the flight direction of the particle and the normal direction of the boundary in the intersection point.

At the end of each batch SIM keeps track of the mean detector signals and the second moments of these signals. Optionally the batch results are stored on a HERMES submission file (IOSIM) for further processing in STATIST.

At the end of run SIM automatically computes the mean values of the detector signals and the statistical errors of these mean values.

No normalization is performed automatically except to the number of source particles. Thus, normalization has to be performed per request of CMD commands at the end of run or off-line using the binary files which can be produced to store the SIM results.

A definition of the contributions (signals) derived by the detectors follows:

(1) Particle Yield Detectors (Types 1-3)

measure the average number of secondary particles of type "prt" generated in energy group IE, angle group IA and region IR1(I) per source particle.

(2) Energy Deposition Detectors (Type 4)

measure the energy deposition [MeV] in 19 channels representing the various schemes of energy deposition, integrated over the region volumes of regions IR1(i). The channels are explained in what follows:

ctype	Description
-------	-------------

- | | |
|---|-----------------------------------|
| 1 | ionisation energy loss of protons |
| 2 | ionisation energy loss of π^+ |
| 3 | ionisation energy loss of π^- |

Note! p, π^+, π^- reaching their transport cutoff energy are assumed to deposit the remaining kinetic energy in place. Captured π^- mesons, however, deposit their energy through the collision products and nucleus excitation.

- | | |
|---|-----------------------------------|
| 4 | ionisation energy loss of μ^+ |
| 5 | ionisation energy loss of μ^- |

Note! stopped μ 's are assumed to decay. We take $(E + m_0c^2)/(m_0c^2) * 33.328$ MeV to be the energy of the electron (positron) and deposit this amount in place. For μ^- we subtract .511 MeV representing the mass of a captured e^- .

- | | |
|---|---|
| 6 | Energy deposited by evaporation deuterons |
| 7 | Energy deposited by evaporation tritons |
| 8 | Energy deposited by evaporation He^3 ion |
| 9 | Energy deposited by evaporation α 's. |

Note! Since this particles are transported in HETC, because of their low range we assume that they deposit their kinetic energy in place.

- | | |
|----|--|
| 10 | The recoil energy of the struck nucleus is also deposited in place |
| 11 | kinetic energy of protons escaping from geometry |
| 12 | kinetic energy of neutrons escaping from geometry |
| 13 | kinetic energy of π^+ escaping from geometry |
| 14 | kinetic energy of π^- escaping from geometry |
| 15 | kinetic energy of μ^+ escaping from geometry |
| 16 | kinetic energy of μ^- escaping from geometry |

Note! The energy of escaping particles is measured in order to provide an impression of the energy loss from the target and must not be taken into account when the total deposition energy is measured.

- | | |
|----|---|
| 17 | Kinetic energy of neutrons which are discarded (and optionally submitted for further treatment) below the neutron cutoff energy |
|----|---|

- 18 Total energy of π^0 (optionally submitted for further treatment in EGS4)
- 19 The excitation energy of struck nuclei (optionally submitted for further treatment in NGEM and EGS4)

Note! Since this energy contributions cannot be handled by HETC itself they are measured as source energy so far. Normally the spread of this energy can be handled by MORSE, EGS4, and NDEM.

(3) Scintillation Light Detector (Type 5)

Detector Type 5 measure the "visible energy" deposited in [MeV] integrated over the volumes of regions IR1(i). This detector is also divided into 19 contribution channels. Channels 10 to 19 measure energy as described above. There are no means available in HETC and SIM to convert these channels to the visible energy fraction. The channels 1 to 9, however, can be converted using Birk's law. We have tabulated the scintillation light as a function of particle energy for protons, charged pions and muons and for the ions resulting from evaporation. The tables are generated and stored along with the setup of energy range tables in HETC.

$$L(E) = S_{med} \int_0^E \frac{1}{1 + k_{Bmed} dE/dx(\epsilon)} d\epsilon \quad (2.1)$$

For charged particles in flight

$$L(E) - L(E_c)$$

is deposited, where E is the energy of the particle at the start of the flight path and E_c is the energy at the end. Particles below transport cutoff are assumed to deposit their energy without escaping the region, so L(E) can be used. (For details see APPENDIX A.2.4).

(4) Forward Current Detector (Type 6)

This detector measures the number of particles crossing the specified boundaries from region IR1(I) to region IR2(I) (i.e. forward) per source particle in energy group IE and angle group IA. It may be renormalized to [A] multiplying it with the source current and normalizing it to the area of the specified boundaries.

The dimension is 1/source particle.

(5) Forward Flux Detector (Type 7)

This detector measures the (surface integrated) flux of particles crossing the specified boundaries from region IR1(I) to region IR2(I) (i.e., forward) per source particle in energy group IE and angle group IA. The contribution of the particle is taken as $wt/[\cos(u, N_s)]$

where wt is the statistical weight of the particle

u is the particle flight direction

N_s is the surface normal direction at boundary crossing point.

The dimension is 1/source particle.

(6) Tracklength Detector (Type 8)

This detector measures the (volume integrated) particle flux [cm] by tracklength of particle type "prt" in region IR1(i), energy group IE and angle group IA. As mentioned above the energy binning is splitting down the flight path according to the appropriate energy groups. The dimension is cm/source particle.

(7) Energy Transfer across Boundaries (Type 9)

This quantity is measured in [MeV] on all specified boundaries from regions IR1(i) to regions IR2(i), separately for 7 particle types

- 1 protons
- 2 neutrons
- 3 π^+
- 4 π^-
- 5 μ^+
- 6 μ^-
- 7 ions (not yet implemented)

The dimension is MeV/source particle.

2.3.3 The Statistics Module STATIST

STATIST has been created to combine the results of correlated Monte-Carlo runs. It reads the detector results stored on HERMES submission files at the end of each Monte-Carlo batch. Arithmetical and logical operations can be applied on the loaded data to derive additional information from the detector output. Detector logic like coincidence or veto can be simulated. Finally a set of physical quantities is available for each batch. Frequency distribution functions of these variables can be requested. A set of statistical moments is computed for each frequency distribution function including the mean value, the fractional standard deviation, scewness, and kurtosis based on empirical estimate statistics. Also the mean value, the f.s.d., the mode and the median of the grouped data are shown in a histogram. The mean and the f.s.d. of a Gaussian fit can be obtained in the range $\mu_e \pm 2\sigma_e$, where μ_e , σ_e are the empirical moments.

The correlation between variables can be shown by means of scatter plots. A linear regression procedure is provided to measure the correlation of variables. The variable set derived from loaded detector results can be stored on a file for further processing in more advanced statistics modules like SAS /14/.

Since STATIST produces plots, it is not totally machine independent. We offer routines which produce line printer plots to overcome this problem (see APPENDIX D). The interfacing requirements for a user provided graphic software can be found also in APPENDIX D.

2.3.4 The Command Input Interpreter CMD

Since HERMES consists of a number of independent programs working on the same problem, it would be helpful to have a problem oriented standard input to avoid inconsistent and redundant input coding. Furthermore this input should not require fixed column formats which always cause troubles like misplaced input items or dummies for data actually not needed. We also would like to increase the legibility of the input; this means input items should be associated with names identifying their meaning.

We made a very first approach to provide such a tool called CMD for input processing. Currently it is used to prepare geometry input for the Monte Carlo programs, analysis input for HETC and

the input for the STATIST code. CMD input tools for the material data input of the Monte-Carlo codes and other parts will follow in subsequent releases of HERMES.

CMD is organized in syntactic levels. The lowest level allows free column input of constants. In the second level we have implemented a syntax of user variable and array definition. The highest level is the most problem oriented one and provides command and statement syntax.

2.4 Planned Future System Extensions

2.4.1 Fast Time-Dependent Neutron History Module DYMO

The code DYMO (Dynamical Model) was specially designed to calculate the dynamics of sampling calorimetry, that means the time dependence of the various nuclear processes of the low-energy neutron component ($E_{kin,n} \leq E_{cut} \leq 20$ MeV), released by the HETC module.

In depleted uranium-scintillator calorimeters these are mainly the number of fast fission processes, the number of fast fission produced secondary neutrons, the number of neutron capture processes (mainly in DU, of minor importance for calorimetry also in hydrogen) and the number of recoil processes in homogeneous materials (like scintillators TMP/TMS liquids or wrapping papers). Most important is the release of neutron kinetic energy to the protons in the read-out medium. Though the 3-dimensional MORSE-CG module can be operated in a time dependent mode (which is more CPU time consuming, of course), the separate code DYMO /15/ was developed.

DYMO operates in the homogenized (and infinite) material mixture of the calorimeter considered. The simplified use of a homogeneous (zero dimensional) mixture (called material mix, expressed in atoms/cm³, corresponding to the thicknesses used) is justified by the fact that the mean free path for several MeV neutrons is in the range of 10 to 50 mm, which is quite large compared to the usual plate thicknesses of several mm's only used in sampling calorimeters.

DYMO uses small steps in time (e.g. 0.1 ns) to evaluate the number of neutrons being affected either in their energy or lost by fission capture. After each step in time, all neutron energy groups are updated in content according to the group to group transfer probability $w_{gg'}(E_g)$:

$$W(E_g \rightarrow E_{g'}, \Delta t) = \rho \cdot w_{gg'}(E_g) \cdot \Delta x(E_g, \Delta t) \quad (2.2)$$

with ρ = effective density of the mix and

$$\Delta x = \sqrt{(2E_g/m_n)} \cdot \Delta t = \text{fictitious path of neutrons} \quad (2.3)$$

with masses m_n and kinetic energy E_g . The time step Δt has to be chosen so small that the reaction probability W is below a certain value (e.g. 0.1) even for the highest neutron energy group and the highest group-to-group transfer probabilities $w_{gg'}$. Otherwise the result would depend on the choice of E_g and of the reaction type.

DYMO is therefore not a Monte Carlo but a deterministic code, iterating in time steps up to a certain limit, typically 1 μ sec. The group-to-group transfer probabilities are extracted from the same large evaluated nuclear data libraries used by the MORSE-CG module.

The updating in time includes the gain of neutrons by fast fissions (secondary or even tertiary fissions; the primary fission was the high-energy fission, calculated with the HETC module). For the fast fission spectrum a Maxwellian distribution with $kT=1.33$ MeV is used up to now.

Since DYMO keeps continuously track of the neutron spectrum versus time and takes into account the transition rates to all neutron induced reactions specified, one can also determine the abundances of the various nuclear γ sources (neutron capture, fast fissions, inelastic and nonelastic neutron induced reactions) versus time by using coupled neutron- γ cross-section libraries.

For DYMO a surface loss correction can be taken into consideration, accounting for the fractional loss of those recoil protons, which gain enough recoil energy to escape from the scintillator layer (or other homogeneous read-out media) into the absorber (or other nonactive media). Since there are no real layers in DYMO, this correction applies only for the mean.

The saturation effect of the read-out material is included in the evaluation of the visible proton recoil energy. As discussed in Section 2.3.2, Equ. (2.1) the saturation is described by the KB parameter in the Birks formula. The energy of the recoil proton is determined by subtracting the neutron energy (given as the mean of the corresponding energy group) after the time step Δt from the energy before the time step, if the transfer probability for elastic scattering on hydrogen was used.

2.4.2 The Myon Transport Module MEGS-CG

A Monte-Carlo technique is provided to investigate the energy loss of muons in sampling calorimetric devices. Two Monte-Carlo codes have been combined to calculate the muon transport with δ electrons-, bremsstrahlung- and pair production and the transport of these secondaries (EGS4, Section 2.2.3). The passage of a muon travelling through many interleaved layers of absorber (high Z) and detector (low Z) sheets results in δ ray production, bremsstrahlung and e^+, e^- pair production. These effects are energy and material dependent and are in general quite different for absorber and detector layers. δ electrons created in the absorber can deposit part of their energy in the detector material and, vice versa, δ electrons created in the detector can deposit part of their energy in the absorber material. Furthermore bremsstrahlung and pair production give rise to electromagnetic showers, which can extend over several layers.

For the transport of the primary muon we use the Monte-Carlo code MUDEX of Lohmann, Kopp and Voss, CERN. This program randomly produces δ electrons, bremsstrahlung, γ 's and e^+, e^- pairs.

Interactions with energy transfers below a given threshold are taken into account by calculating a mean value for this part of the energy loss. The energy is assumed to be deposited locally. In a second step, using EGS4, each of the produced higher energetic δ rays, bremsstrahlung and particle pairs is followed through the calorimeter. We call the new code obtained by linking MUDEX to EGS4 MEGS-CG /16/. Such a Monte-Carlo code offers the possibility to calibrate sampling calorimeters with the help of muons.

2.4.3 The Hadron Cascade Code FLUKA

At the higher energies of present interest, i.e. hundreds of GeV, the most reliable model now available seems to be a multi-chain fragmentation model of hadron-nucleus collision developed and implemented into the Monte Carlo code FLUKA by J. Ranft et al. /17/ following the work of A. Capella et al. /18/.

We converted the FORTRAN IV version of FLUKA86 into a FORTRAN77 version for being adapted to the HERMES system and the CG-geometry CG-L2.

An analysis package on SIM basis has been written which is analog to SIM-HETC analysis routines. Coupling to the HERMES submission file is being prepared.

2.5 Structure of the HERMES Submission File

The modular structure of the HERMES code system requires the introduction of a convention for all inter-code communication data.

We designed a simple sequential file structure for this purpose. The data needed by the programs to communicate are

- physical properties of particles which are to be transported through the physical domain of more than one Monte-Carlo-code,
- trigger events like events like "start of run", "end of run", "start of batch (event)" and "end of batch (event)", which help to identify correlated data, and
- detector output data.

For each of the above data items one or two records are written on the HERMES submission file. All records begin with a record name (CHARACTER*8) and a the record type (integer number) to identify the meaning of the record. The following Tables 2.4-8 illustrate the structure of the records currently used.

Table 2.4
Format of trigger event records

item	name	type	description
1	VERB	char*8	record name
2	KEY	integer	situation key record type
3	ISEED	integer	current random seed
4	IRUN	integer	problem number
5	IBAT		current batch number
6	IHIS		current history number
7	NNEU	integer	no. of neutrons submitted (HET)
8	NPIO		no. of π^0 submitted (HET)
9	NNUC		no. of excited nuclei submitted (HET)
10	JOB	char*8	Time stamp to identify the job
11	DATE		
12	TIME		
13	dummy		dummy words to get same record
14	dummy		length of IBM and CRAY/XMP
15	dummy		

Table 2.5
Types of trigger event records

VERB	KEY	situation
SOR pgm	-1	Start of run
SOB pgm	-2	Start of batch
EOB pgm	-3	End of batch
EOR pgm	-4	End of run

Table 2.6
Format of particle submission records

item	name	type	description
1	VERB	char*8	record name
2	KEY	integer	=0 for all particle submission records
3	ISEED	integer	current random seed
4	A	real	struck nucleus mass number
5	Z		struck nucleus charge number
6	E*		struck nucleus excitation energy [MeV]
7	TYP	real	particle type
8	X		
9	Y		particle site
10	Z		
11	U		
12	V		particle direction cosines
13	W		
14	E		particle energy [MeV]
15	WT		particle statistical importance

Table 2.7
Types of particle submission records

VERB	KEY	TYP	description
N pgm	0	1.	n (below cut)
GAM pgm	0	7.	γ
PIO pgm	0	3.	π^0
NUC*pgm	0	14.	excited nucleus
E+ pgm	0	8.	e^+
E- pgm	0	9.	e^-
MU+ pgm	0	5.	μ^+
MU- pgm	0	6.	μ^-

Table 2.8
Format of (detector) data array submission records

item	name	type	description
1	VERB	char*8	record name=data array name
2	KEY	integer	record type
3	LEN	integer	total data array length
4	N1		array dimensions
5	N2		
6	N3		
7	V1	real	arbitrary real information
8	V2		
9	V3		
10	JOB	char*8	time stamp of the
11	DATE		generating job
12	TIME		
13	dummy		not used
14	dummy		
15	dummy		

The above data array control record is followed by a data record of length LEN, representing the data array VERB(N1,N2,N3). For integer arrays KEY must be 1, for real arrays KEY must be 2.

2.6 References Chapter 2

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3. INPUT DESCRIPTION AND OPERATION OF THE HERMES CODE SYSTEM

3.1 Guide to HERMES Operation

The organization scheme of the HERMES system is illustrated in Fig. 2.1 of Chapter 2.

The operation of HERMES proceeds in up to 7 steps:

(1) Generation of the Geometry Description

Programs:	CMD, CG-L2, GEOTEST routines
Input:	CG-L2 input in CMD language, additional CMD commands to test the geometry setup (see Section 3.9)
Output:	CG-L2 input file in card image format, test results (tables and viewgraphs)

Detailed description: Section 3.6

(2) Run of HETC with Online Analysis

Programs:	HET/KFA2, SIM, CG-L2, CMD, STATLIB
Input:	- CMD-commands defining the detectors for online analysis, specification of materials, the particle source, and the model options - CG-L2 geometry input in card image format - a binary file containing nucleon cross-sections and evaporation data
Output:	- a print file containing the HET run log and the online analysis results (from SIM) - a HERMES submission file containing low energy neutrons, π^0 and the resi- dual nuclei after INCE - a HERMES submission file containing the SIM detector results for each batch

Detailed description: Sections 3.2 and 3.7

(3) MORSE Transport of Low Energy Neutrons

Programs: MORSE/KFA, CG-L2

Input:

- specification of model options, particle source, materials used, detectors
- CG-L2 input in card image format
- a submission file containing the source neutrons (from HET)
- a cross-section library (e.g. EPR) containing the neutron (and γ) group cross-sections

Output:

- a print file containing the MORSE run log, and detector results
- a HERMES submission file containing γ 's from (n, γ) processes
- a HERMES submission file containing the detector results of each batch

Detailed description: Section 3.3

(4) Generation of γ Sources by NDEM from de-Excitation of Residual Nuclei

Programs: NDEM

Input:

- a HERMES submission file containing the nuclei to be de-excited
- a library containing the decay modes (level densities) of excited nuclei
- specification of model options

Output:

- a HERMES submission file containing the γ 's generated
- a print file

Detailed description: Section 3.5

(5) Preparation of Material Data to be Used by EGS

Programs: PEGS4

Input:

- specification of the materials to be used
- a data file containing data for the photoelectric effect, pair production and Rayleigh scattering (Storm and Israel)
- a data file containing the Rayleigh scattering form factor

Output:

- a cross-section file to be used in EGS4
- miscellaneous printed output, the generated data and viewgraphs

Detailed description: Section 3.4.2

(6) Run of the EGS Code

Programs: EGS4/KFA1, STATLIB, CG-L2

Input:

- specification of the model options, material names, and the particle source
- CG-L2 geometry input in card image format
- a HERMES submission file which may contain π^0 , e^- , e^+ or γ
- cross-section data generated by PEGS

Output:

- a print file containing the EGS4 run log and detector results
- a HERMES submission file containing the detector results of each batch

Detailed description: Section 3.4

Note! EGS4 may be used several times in the same problem picking up EM sources of different origins, namely π^0 decay, residual nucleus de-excitation, (n, γ) reactions etc.

(7) Final Analysis of the Combined HET, MORSE and EGS Results

Programs: STATIST, CMD

Input:

- a command file (CMD) specifying the analysis to be made
- one or more HERMES submission files containing detector results from Monte Carlo runs

Output:

- viewgraphs of frequency distributions, scatterplots, tables of statistical moments
- a binary file containing processed results of each batch for use in SAS or other statistical procedures

Note! STATIST can be used after each Monte Carlo run to analyse intermediate results

Detailed description: Section 3.8

3.2 HET Input Description

HET input/output is illustrated in Fig. 3.1.

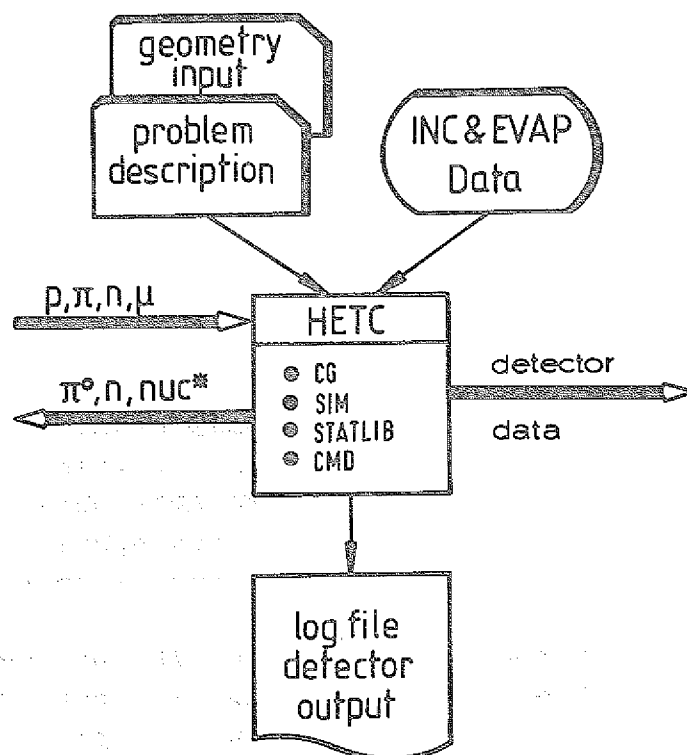


Fig. 3.1: HET input-output

HET requires up to three input files:

- Unit 5 is used for primary input as described below.
- Unit INSUB is used for external source particles on a HERMES submission file
- Unit INGEO is used for geometry-description input as described in Section 3.6.
- Unit IBERTP is used for INC cross-sections and EVAP data.

The primary input is read from unit 5. First CMD is called to read in the detector definitions for online analysis. This part of input must be terminated by the RUN command. Next to this HET reads in a single namelist record &RUN containing the physical parameters of the problem to be solved. The variables used, their default values and meanings are given in Table 3.1. (CMD see Section 3.7.1).

Finally CMD is called again to provide final processing of the analysed data. This CMD input section must be terminated with the EXIT command.

Table 3.1

VARIABLE	Default	Description
IRUN	1	Identification number for the run
MAXCAS	2	Number of source events per batch
MAXBCH	2	Number of batches to be run
ISEED	123456789	Starting random seed
INSUB	0	Unit number for external source file in HERMES format.
IOSUB	0	Unit number of particle submission file in HERMES format.
INGEO	5	Unit number of geometry input data read by JOMIN
IOGEO	6	Unit number for JOMIN print file
NHSTP	0	Unit number for history file
IBERTP	-20	Unit number for INC EVAP data file if >0 nucleon pion transport only if <0 nucleon pion muon transport
EMIN(1)	15.0	Transport energy cut for protons [MeV]
EMIN(2)	15.0	Transport energy cut for neutrons [MeV]
EPICUT	1.0	Transport energy cut for pions [MeV]
EMUCUT	1.0	Transport energy cut for muons [MeV]
EMAX		Maximum particle energy [MeV]
N1COL	0	if >0 stop history after second generation useful in debugging

Table 3.1 continued

VARIABLE	Default	Description
NPIDK	1	π^- capture option if =1 π^- decay at cut-off energy if =0 π^- are captured at cut-off energy
KEYDK	0	Pion decay-in-flight option if =0 weighted decay to increase number of muons if =1 analog decay
NBOGUS	1	Option to correct residual nucleus excitation with the recoil energy
NSPRED	0	Option to allow small angle multiple Coulomb scattering of charged source particles
NWSPRD	0	Option to report small angle multiple Coulomb scattering to history file
NSEUDO	1	Option to report pseudo events to history file
ISTRAG	0	Option to perform charged-particle range straggling.
IANG	1	Option to allow non-isotropic evaporation
IFISS	0	Option to allow high energy fission
IBO	1	Option to use automatic B_0 in evaporation
B0	8.0	Level density parameter to be used in evaporation
IELAS	0	Elastic scattering option if =0 no elastic scattering performed (except with ^1H) if =1 neutron elastic scattering switched on if =2 neutron and proton elastic scattering switched on
MXMAT		Number of media used (<17, except voids)
SO(M)	1.0	Detector efficiency of light detectors in medium M
KB(M)	0.0	Saturation parameter of light detectors in medium M

Table 3.1 continued

VARIABLE	Default	Description
DENH(M)		Numerical density of ^1H in medium M [atoms/cm ³ *1.E24]
NEL(M)		Number of elements other than ^1H in medium M
A(L,M)		Mass number of element L in medium M
ZZ(L,M)		Charge number of element L in medium M
DEN(L,M)		Numerical density of element L in medium M [atoms/cm ³ *1.E24]
LHI	0	Switch for heavy ion sources use 1 for d , 2 for t 3 for He3 , 4 for He4 5 for arbitrary heavy ion
HIA(5)		mass of arbitrary heavy ion (<20)
HIZ(5)		charge of arbitrary heavy ion
HIBE(5)		binding energy of arbitrary heavy ion
TIPSR	0	Source particle type use 0. for protons , 1. for neutrons, 2. for π^+ , 4. for π^- 5. for μ^+ , 6. for μ^- 7. for d(LHI=1) , 8. for t(LHI=2) 9. for ^3He (LHI=3), 10. for ^4He (LHI=4) 11. for heavy(LHI=5)
XSR YSR ZSR	0	Starting coordinates for source particles
XSIG YSIG ZSIG		Standard deviations for XSR YSR ZSR
USR VSR WSR	0.0	Direction cosines for source particles
	0.0	Default refers to isotropic
	0.0	distribution
ESR		Energy of source particles [MeV]
WTSR	1.0	Statistical weight of source particles

Note! source particle parameters may be omitted for external sources; e.g. when INSUB>0

Note! Geometry input is read from unit INGEO. If INGEO=5, then geometry input must follow the HET/KFA2 NAMELIST record on the same file.

3.3 MORSE-CG Input Description

MORSE-CG input/output is illustrated in Fig. 3.2.

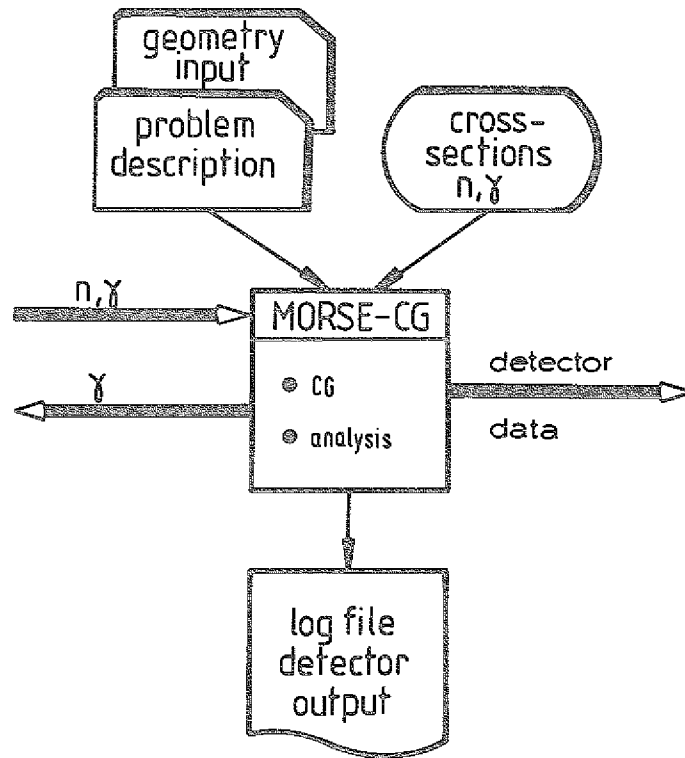


Fig. 3.2: MORSE-CG input-output

MORSE-CG requires up to four logical units for input:

- Unit 5 is used for primary input as described below.
- Unit IHETTP is used for external source particles on a HERMES submission file
- Unit INGEO is used for geometry description as described in Section 3.6.
- Unit IXTAPE is used for binary cross-section data (e.g. the EPR library)

MORSE-CG requires up to six logical units for output:

- Unit 6 is used for primary output
- Unit IOGEO is used for geometry description
- Unit IORSY is used for detector response data in RSYST structure /1/
- Unit IOCNT is used for MORSE counters
- Unit ISIMTP is used for the detector response data file on a HERMES submission file
- Unit IGAMTP is used for γ rays of (n, γ) coupled MORSE runs on a HERMES submission file

The primary MORSE-CG input is read from unit 5. Input to MORSE-CG is written in card image or REA format as described below (Section 3.3.1).

Geometry input is read from unit INGEO. If INGEO=5, geometry input must follow the random walk module input in the same file.

(1) Randwalk Module Input Instructions

The input read by subroutine INPUT1 is as follows:

Card A	FORMAT (20A4)
Title card.	
(Any character other than blank or alphameric in column 1 will terminate the job.)	
Card A2	FORMAT (*)
IOCNT	Unit number for counter output
IORSY	Unit number for RSYST /1/ output
INGEO	Unit number for geometry input
IOGEO	Unit number for geometry output
Card B	FORMAT (*)
NSTRT	Number of particles per batch; unused in case of particle input from a HERMES submission file
NMOST	Maximum number of particles allowed in the bank(s) may equal NSTRT if no splitting, fission, and secondary generation and no use of HERMES submission file otherwise NMOST > maximum number of particles per batch
NITS	Number of batches
NQUIT	Number of sets of NITS batches to be run without subroutine INPUT; unused, must be 1
NGPQTN	Number of neutron groups being analyzed to simulate energy cut).
NGPQTG	Number of gamma ray groups to be analyzed.
NMGP	Number of primary particle groups for which cross sections are stored; should be same as NGP (or the same as NGG, when NGP = 0) on Card XB read by cross-section module
NMTG	Total number of groups for which cross-sections are stored; should be same as NGP+NGG as read on Card XB read by cross-section module

NCOLTP Unit number for collision file; to be set greater than zero if a collision file is desired; the collision file is written by the user routine BANKR; not used in this version of MORSE-CG

IADJM set greater than zero for an adjoint problem

AXTIM unused variable

MEDIA Number of cross section media; should agree with NMED on Card XB read by the cross-section module.
For using the volume light detector, MEDIA must be number of media + one extra medium for each light response (containing ^1H)

MEDALB Albedo scattering medium is absolute value of MEDALB;
if MEDALB = 0, no albedo information to be read in,
MEDALB < 0, albedo only problem - no cross sections are to be read,
MEDALB > 0, coupled albedo and transport problem.

Card C FORMAT (*)

ISOUR Source energy group if > 0,
if ISOUR < 0 or ISOUR = 0 and NGPFS $\neq$ 0,
SORIN is called for input of Card E1 and E2.

NGPFS Number of groups for which the source spectrum is to be defined.
if ISOUR $\leq$ 0, NGPFS $\geq$ 2.

ISBIAS No source energy biasing if set equal to zero; otherwise the source energy is to be biased, and Cards E2 are required

NOTUSD An unused variable

WTSTRT Weight assigned to each source particle.

EBOTN Lower energy limit of lowest neutron group (eV) (group NMGP)

EBOTG Lower energy limit of lowest gamma-ray group (eV) (group NMTG)

TCUT Age in sec at which particles are retired;
if TCUT = 0, no time kill is performed.

VELTH Velocity of group NMGP when NGPQTN > 0; i.e., thermal neutron velocity (cm/sec) where
 $\text{VELTH} = \text{SQRT}((\text{WTS}(\text{I}) + \text{WTS}(\text{I}+1))/2) \cdot 1.383\text{E}+6.$

TABLE 3.2
Sample Group Input Number for Some
Representative Problems*

Case A - Neutron Only Cross Sections (22 groups)
Case B - Gamma Ray Only Cross-Sections (18 groups)
Case C - Neutron-Gamma Ray coupled Cross-Sections
(22-18 groups)

Input Variable	Case A Top 14 groups	Case B Top 17 groups	Case C Neutrons only Top 14 groups	Case C gamma- rays only Top 17 groups	Case C Neutron- gamma-rays Top 14 Neutron Top 17 gamma-rays

MORSE Input					
NGPQTN	14	0	14	0	14
NGPQTG	0	17	0	17	17 Card B
NMGP	22	18	22	18	22 variables
NMTG	22	18	22	18	40
NGP	22	18	22	0	22+
NGG	0	0	0	18	18 Card XB
INGP	22	18	40	40	40 variables

*For cross sections with full downscatter, NDS=NGP, NDSG=NGG, INDS=INGP, and ITBL=number of downscatters + number of upscatters +3.

Usually, ISGG=number of upscatters + 4; i.e. NUS+4.

+Must be = to total number of neutron groups in the data, otherwise it picks up gammas from wrong location.

Card D1 FORMAT (*)

 XSTRT

 YSTRT } Coordinates for particles

 ZSTRT

 AGSTRT Starting age for source particles

 UINP Source particle direction cosines, if all

 VINP } are zero, isotropic distribution is assumed

 WINP

Note! Source data on Cards C and D1 will be overridden by
using a HERMES submission file

Card D2 FORMAT (*)
 IHETTP Logical unit number of HERMES submission
 file, if greater zero
 ISIMTP Logical unit number of response data file
 in HERMES submission file structure, if
 greater zero
 IGAMTP Logical unit number of gamma ray particle
 file in HERMES submission file structure,
 if greater zero

Cards E1 FORMAT (*)
 (omit if ISOUR on Card C > 0 or if ISOUR = NGPFS = 0)
 NGPFS values of FS, where FS equals the unnormalized frac-
 tion of source particles in each group.

Cards E2 FORMAT(*)
 (omit if ISOUR > 0 or if ISOUR <= 0 and ISBIAS = 0)
 If ISBIAS > 0, NGPFS values of BFS, the relative importance
 of a source in group I is required.

Cards F FORMAT(REAG)
 NMTG values of ENER, the energies (in eV) at the upper edge
 of the energy group boundaries.

Note! The lower energies of groups NMGP and NMTG were read on
 Card C

Card G FORMAT(*)
 (omit if NCOLTP on Card B <= 0)
 NHISTR Logical unit number for the first collision
 file
 NHISMX The highest logical unit number that a collision
 file may be assigned to.
 NBIND(J),J=1,36 An index to indicate the collision parame-
 ters to be written on file
 NCOLLS(J),J=1,13 An index to indicate the types of colli-
 sions to be put on file

Note! unused in this MORSE-CG version

Note! See Table 3.3 for information concerning NBIND

Table 3.3
Variables that may be Written on Tape (NBIND)

J	Variable*	J	Variable
1	NCOLL	19	WTBC
2	NAME	20	ETAUSD
3	IG	21	ETA
4	U	22	AGE
5	V	23	OLDAGE
6	W	24	NREG
7	X	25	NMED
8	Y	26	NAMEX
9	Z	27	WATEF
10	WATE	28	BLZNT
11	IGO	29	BLZON
12	UOLD	30	VEL(IG)
13	VOLD	31	VEL(IGO)
14	WOLD	32	TSIG
15	XOLD	33	PNAB
16	YOLD	34	NXTRA
17	ZOLD	35	EXTRA1
18	OLDWT	36	EXTRA2

* These variables are defined in tables 4.4 and 4.5 of Ref. /2/

Card H FORMAT(Z12)
RANDOM Starting random number

Card I FORMAT(*)
NSPLT Index indicating that splitting is allowed
 if > 0.
NKILL Index indicating that Russian roulette is
 allowed if > 0.
NPAST Index indicating that exponential transform
 is invoked if > 0
NOLEAK Index indicating that non-leakage is in-
 voked if > 0.
IEBIAS Index indicating that energy biasing is
 allowed if > 0.
MXREG Number of regions described by geometry
 input (will be set to one if <= 0.
MAXGP Group number of last group for which Russian
 roulette or splitting or exponential trans-
 form is to be performed. For adjoint,
 set = NMTG or overstoring result.
IOCNT option for Region output

Card I2 FORMAT(REAI)
(only if MXREG < 0)
 geometry regions

Card I3 FORMAT(REAI)
(only if MXREG = -2)
 MORSE regions

Card J FORMAT(*)
 (omit if NSPLT + NKILL + NPAST =)
 NGP1 From energy group NGP1 to energy group NGP2,
 NDG inclusive, in steps of NDG and from region
 NGP2 NRG1 to region NRG2, inclusive, in steps of
 NRG1 NDRG, the following weight standards and
 NDRG path-stretching parameters are assigned.
 NRG2 If NGP1=0, groups 1 to MAXGP will be used;
 if NRG1=0, regions 1 to MXREG will be used
 (both in steps of 1). Usually NDG=1 and NDRG=1.
 WTHIH1 Weight above which splitting will occur
 WTLOW1 Weight below Russian roulette is played
 WTAVE1 Weight given those particles surviving
 Russian roulette
 PATH Path-length stretching parameters use in
 exponential transform (usually $0 \leq \text{PATH} < 1$)

Note! The above information is repeated until data for all groups and regions are input

End Card J with negative value of NGP1 (i.e. -1 /)

Note! In runs with quasi analog calculation of splitting, fission and gamma production WTHIH1, WTLOW1 and WTAVE1 must be 1.

Card K FORMAT(REAG)
 (omit if IEBIAS on Card I ≤ 0)
 ((EPROB(IG,NREG), IG=1, NMTG), NREG=1, MXREG)
 Values of the relative energy importance of particle
 leaving a collision in region NREG.

Card L FORMAT(*)
 NSOUR Set ≤ 0 for a fixed source problem; otherwise the source if from fissions generated in previous batch.
 (NSOUR=1 for k_{eff} calculations)
 MFISTP Index for fission problem, if ≤ 0 no fissions are allowed.
 NKCALC The number of the first batch to be included in the estimate of k; if ≤ 0 no estimate of k is made.
 NORMF The weight standards and fission weights are unchanged if ≤ 0 ; otherwise fission weights will be multiplied, at the end of each batch, by the latest estimate of k and the weight standards are multiplied by the ratio of fission weights produced in previous batch to the average starting weight for the previous batch. For time-dependent decaying systems, NORMF should be > 0 .

Card M FORMAT(REAG)
 (omit if MFISTP on Card L <= 0)
 (FWLO(I), I=1,MXREG) values of the weight to be assigned
 to fission neutrons.

Card N FORMAT(REAG)
 (omit if MFISTP on Card L <= 0)
 (FSE(IG,IMED), IG=1,NMTG), IMED=1,MEDIA)
 The fraction of fission-induced source particles in group
 IG and medium IMED

Card O FORMAT(REAG)
 (omit if NGPQTN=0 or NGPQTG=0, i.e. include if coupled
 neutron-gamma ray problem)
 ((GWLO(IG,NREG), IG=1,NMTG), NREG=1,MXREG)
 Values of weight to be assigned to the secondary particles
 being generated.

(2) Cross-Section Module Input Instructions

The input read by subroutine XSEC is as follows:

Card XA FORMAT(20A4)
 Title card for cross sections. This title is also written
 on file if a processed file is written.

Card XB FORMAT(*)

NGP	The number of primary groups for which there are cross-sections to be stored. Should be same as NMGP read on Card B in randwalk module
NDS	Number of primary downscatters for NGP (usually NGP). Includes $\sigma(g\ g)$ in case of downscatter cross-section matrix.
NGG	Number of secondary groups for which there are cross sections to be stored
NDSG	Number of secondary downscatters for NGG (usually NGG)
INGP	Total number of groups for which cross- sections are to be input.
ITBL	Table length, i.e., the number of cross- sections for each group (usually equal to number of downscatters + number of up- scatter + 3)
ISGG	Locations of within-group scattering cross- sections (usually equal to number of up- scatter + 4).

NMED Number of media for which cross-sections are to be stored; should be same as MEDIA read on Card B in random walk module. for using the volume light detector NMED must be number of media + extra medium for each light response

NELEM Number of elements for which cross-sections are to be read.

NMIX Number of mixing table entries (must be ≥ 1).

NCOEF Number of coefficients for each element, including P0.

NSCT Number of discrete angles (usually $[NCOEF/2]$)

ISTAT Flag to store Legendre coefficients if greater than zero. (must be = 1 for point detector)

Card XC FORMAT(*)

IRDSG Switch to print cross-sections as they are read, if > 0

ISTR Switch to print cross-sections as they are stored, if > 0 .

IFMU Switch to print intermediate results of mean value calculation, if > 0

IMOM Switch to print moments of angular distribution, if > 0

IPRIN Switch to print angles and probabilities, if > 0

IPUN Switch to print results of bad Legendre coefficients, if > 0
if = 1 message in case the first moment is wrong; in case P2 and P3 are wrong, they are ignored and MORSE calculates just P1.

IDTF Switch to signal that input format is DTF-IV format, if > 0 ; otherwise ANISN format is assumed

IXTAPE Logical unit number if binary cross-section file, set equal to 0 if cross-sections are read from cards. if negative, then the processed cross-sections and other necessary data from a previous run will be read; in this case (IXTAPE <0) no cross-sections from cards and no mixing cards may be input and switches are ignored
the absolute value of IXTAPE is the logical unit number

JXTAPE Unit number of a processed cross-section file to be written. this processed file will contain the title card, the variables from common LOCSIG and the pertinent cross sections from blank common

IO6RT Unit number of a point cross-section file in O6R format

IGQRT Last group (MORSE multigroup structures) for which the O6R point cross-sections are to be used (must be $\leq$ NMGP)

IRBA if IRBA=0, EPR-Library or 99-group Library
 =1, cross sections generated by RSYST are input. these cross-sections do not contain the factor $(2L+1)$, so they are multiplied by $(2L+1)$ in MORSE
 =2, ANISN-formatted Library, e.g. LASL

ICRO Switch to print neutron to gamma transfer cross-sections, if > 0

IBAD Switch to print bad moments, if > 0

Card XD FORMAT(REAL)

(omit if IXTAPE ≤ 0)

Element identifiers for cross section file. If element identifiers are in same order as elements on file, the efficiency of the code is increased due to fewer file rewinds.

Card XE

(omit if IXTAPE $\neq 0$)

If cross sections are in free-form, a card with ** in columns 2 and 3 must precede the actual data.

ANISN format if IDTF ≤ 0 ; otherwise, DTF-IV format. Cross sections for INGP groups with a table length ITBL for NELEM elements each with NCOEF coefficients are read in.

Note! should not be used in this MORSE-CG version

Cards XF FORMAT(*) MIXING TABLE

(omit if IXTAPE < 0)

NMIX (see Card XB) cards are required.

KM Medium number

KE Element number occurring in medium KM (negative value indicates last mixing operation for this medium). If the medium consists of only one element, this has to be negative, where KE is the cross section numbers read on Card XD in order of the input
 Failure to have a negative value causes code not to generate angular probabilities for that medium.

RHO Atom density of element KE in medium KM

Note! for each medium (scintillator material) for which visible energy is to be calculated (by using the volume light detector) an extra medium containing ^1H must be defined with the same density as the ^1H density in the scintillator material.

Card XG FORMAT(*)
(omit if IO6RT <= 0)
 NXPM Number of point cross-section sets per
 medium found on an O6R-file.
 = 1, total cross-section only
 = 2, total + scattering cross-section
 = 3, total, scattering and ν *fission
 cross section

(3) SAMBO Analysis Input Instructions

The following data are read from cards by subroutine SCORIN:

Card AA FORMAT(20A4)
 Title information - will be immediately output.

Card BB FORMAT(*)
 ND Number of detectors (set =1 if <=0).
 NNE Number of primary particle (neutron) energy
 bins to be used (must be <= NE).
 NE Total number of energy bins (set =0 if <=1)
 if NNE < NE the first NNE-1 energy bins and
 the last one are used. There will be a message
 in the output
 NT Number of time bins for each detector (may
 be negative, in which case the absolute values
 of NT are to be read and used for each detector)
 (set =0 if <=1)
 NA Number of angle bins (set =0 if <=1)
 NRESP Number of energy dependent response func-
 tions to be used (set =1 if <=0)
 NEX Number of extra arrays of size NMTG to be
 set aside (useful, e.g., as a place
 to store an array of group-to-group trans-
 fer probabilities for estimator routines)
 must be >=1 for point detector
 NEXND Number of extra arrays of size ND to be set
 aside (useful, e.g., as a place to
 store detector dependent counters)
 must be = NNE for volume light detector
 IORESP Switch to print responses, if > 0
 IOFLUX Switch to print fluence, if > 0

Card BC	FORMAT(A)
CHOICE	Detector type
	VQ : volume flux detector
	VL : volume light detector
	P : point detector
	SQ : surface flux detector
	SC : surface current detector
	SE : surface energy current detector

Cards CC1-CC5 Detector information

Format:	Card CC1	FORMAT(REAG)
	Card CC2	FORMAT(REAG)
	Card CC3	FORMAT(REAG)
	Card CC4	FORMAT(REAI)
	Card CC5	FORMAT(REAI)

Note! The meaning of these input variables is totally dependent on the detector type that is chosen!

(a) Volume Detector: VQ and VL

CC1	must be :	F 0. ;	
CC2	must be :	F 0. ;	
CC3	must be :	F 0. ;	
CC4		IREG	region number, where the response is to be estimated
CC5	must be :	F 0 ;	for volume flux detector
		IMEDIA	for volume light detector
			medium number of 1H IMEDIA(I) with the same 1H density as in the material of detector IREG(I)

(b) Point Detector: P

CC1		X	X-Coordinates of detector points
CC1		Y	Y-Coordinates of detector points
CC1		Z	Z-Coordinates of detector points
CC4	must be :	F 0 ;	
CC5	must be :	F 0 ;	

(c) Surface Detector: SQ, SC, SE

CC1		U	U,V,W specify the zero degree
CC2		V	direction for the desired detectors, they reference to direction
CC3		W	cosines for angular dependent calculations
CC4		IR1	regions before surface crossing
CC5		IR2	regions after surface crossing

Note! In case of time dependent calculations with volume or surface detectors the lower limit of the fine time bin is set to zero.

Note! In case of the point detector, the distance between the x,y,z points and XSTRT, YSTRT, ZSTRT values and the initial age, AGSTRT, will be used to define the lower limit of the first time bin

Card DD FORMAT(20A4)

Title or units for total responses for all detectors. Will be used in columns 54 through 133 of the title prior to the print of these arrays

Card EE FORMAT(20A4)

Title or units for each total response for all detectors

Card FF FORMAT(REAG)

Response function values. NMTG values will be read on each set of FF Cards. Input order is from energy group 1 to NMTG (order of decreasing energy)

Note! Cards EE and FF are read in the following order:

NRESP sets of (EE - FF) cards

Card GG FORMAT(20A4)

(omit if NE $\leq$ 1)

Units for energy dependent fluence for all detector

Card HH FORMAT(REAI)

(omit if NE $\leq$ 1)

Energy group numbers defining lower limit of energy bins (in order of increasing group number). The NNE (if > 0) energy must be equal NGPQTN: the NE entry must be set to NMGP + NGPQTG for a combined problem, or else NGPQTG or NGPQTN.

Card II FORMAT(20A4)

(omit if absolute value of NT $\leq$ 1)

Units for time dependent total responses for all detectors

Card JJ FORMAT(20A4)

(omit if absolute value of NT $\leq$ 1 or NE $\leq$ 1)

Units for time and energy dependent fluences for all detectors.

Cards KK FORMAT(REAG)

ND Cards must be input

NT values of upper limits of time bins for each detector (in order of increasing time and detector number). The values for each detector must start on a new card. Absolute values of NT only are read if NT is negative. They are then used for each detector

Card LL FORMAT(20a4)

(omit if NA <= 1)

Units for angle and energy dependent fluence for all detectors

Card MM FORMAT(REAG)

(omit if NA <= 1)

NA values of upper limits of angle bins (actually cosine bins; the NAth value must equal one, that means the values have to be specified in increasing order of cosine.

(4) Additional Input

Following is the input to the SAMBO analysis module, input cards for user-written routines

For point detector only : CHOICE=P

For detailed information about using the point detector see Section A.4.2 of Appendix A.

Card P1 FORMAT(*)

IDR(I,1),

I=1,ND

ND values, giving the number of regions,

that are specified for each detector i

IDR = 0 : no specification of regions, i.e. all regions are regarded

IDR > 0 : only those collisions are regarded, that occur in one of the regions defined on Card P2

IDR < 0 : collisions, that occur in one of the regions defined on Card P2, are disregarded

Cards P1 FORMAT(*)

(for each I with IDR(I,1) ≠ 0 a new card with:)

IDR(I,J), Regions for each detector

J=1,IDR(I,1)

For volume light detector only : CHOICE=VL

Card VL1 FORMAT(*)

NUMBR

Number of light response functions

NUMBR sets of following cards must be input

Card VL2	FORMAT(*)
NDETEC	Number of detector using this light response function
S	Scintillation efficiency of this light response (fraction of ionization energy converted to fluorescent light energy)
KB	Saturation parameter of this light response
NEL	Number of elements (other than ^1H) of detector medium
DENH	Atom density of ^1H in detector medium
Card VL3	FORMAT(REAI)
ID	NDETEC region numbers of detectors with same medium and using this light response function
Card VL4	FORMAT(REAG)
ZZ	The NEL charge numbers of elements in this medium
Card VL5	FORMAT(REAG)
A	The NEL mass numbers of elements in this medium
Card VL6	FORMAT(REAG)
DEN	The NEL atom densities of elements in this medium

3.3.1 Implementations in MORSE-CG

MORSE-CG was implemented into HERMES to simulate low energy neutron and gamma transport.

(1) Input and Output Files

To use MORSE in the context of HERMES an interface to HERMES submission files has been included. This allows MORSE to receive particle sources generated by other programs (HET, NDEM or by MORSE itself), to report its results for further processing (in STATIST) and to write generated gamma-rays to be read in other programs as a source.

The analysis code of MORSE generates response data records (in HERMES submission format) at the end of each batch, if chosen. The verbs of all these records are in the following form:

Rxxx@EOB with xxx is the response function number in order as read-in from 001 to 999

If chosen, the generated γ rays will be written on a HERMES submission file to be used in other programs, e.g., EGS4, or in MORSE itself as source particles). Therefore, the energy of the γ rays generated in γ -energy group g will be uniformly distributed between the low and high energy boundaries of this group.

(2) The REA Format

To simplify the input of data arrays, MORSE uses two subroutines REAG and REAI, to input arrays from cards. REAI reads INTEGER arrays, REAG REAL arrays. Each input of an array begins on a new card. To read variable 'A', means the whole array or an array element, in REA format use following syntax: Where 'OP' is an operator given as one character,

' N' is the repetition factor given as INTEGER constant

and ' V' is the value given as REAL constant for REAL variable 'A' or INTEGER constant for INTEGER variables

All elements should be separated by blanks;

Table 3.4
Meaning

OP	N	Value	Meaning
		V	$A(I)=V ; I=I+1$
R	N	V	N-times : $A(I)=V ; I=I+1$
Q	N	M	repeat the last M values N-times
F		V	fill remainder of A with V
A	N	V	N-times : $A(I)=A(I-1)+V ; I=I+1$
M	N	V	N-times : $A(I)=A(I-1)*V ; I=I+1$
/			Skip to next line
S			Skip to next line
;			Terminate input (must be the
T			last 'OP')

A message occurs, when too many or too few values are given.

Examples:

F 0. ; Fill the whole REAL vector with the value 0.
 R 41 1 F 2 ; Vector elements A(1) - A(41) have the INTEGER value 1, the remaining element the INTEGER value 2.
 5. A 4 2. means: A(1)=5., A(2)=7., A(3)=9., A(4)=11., A(5)=13.

(3) Auxiliary Routine NDMMOR for γ -Data Input

This program merges together the HERMES submission files generated by NDEM and MORSE, both containing the produced γ rays. Thus one single run can be made for γ rays produced by high and low energy particles.

The unit numbers used by the program are:

- Unit 10 for the HERMES submission file generated by
 NDEM
- Unit 11 for the HERMES submission file generated by
 MORSE
- Unit 12 for the HERMES submission file generated by
 NDMMOR

3.4 EGS4-HERMES Implementation

To use EGS4 in the context of the HERMES code system, a standard user code has been written, which solves the following tasks:

- Interfacing with HERMES submission files.
This allows EGS4 to accept particle sources generated by other codes (NDEM, MORSE) and to report its results for further processing (in STATIST).
- Interfacing of EGS4 with the combinatorial geometry code CG-L2. By means of this interface EGS4 can share the geometry description used also in HET, MORSE etc.
- A standard analysis was incorporated to measure energy deposition, flux and leakage current by tracklength of particles dependent on type and region.

Since the user code is rather large, problem dependent data (such as NMED, NREG, ECUT, etc.) are read-in from an input file rather than being assigned in the MORTRAN source text. Thus, EGS4 in the version used here can be used like a conventional FORTRAN program. The FORTRAN source of the standard implementation is delivered along with the MORTRAN source.

EGS newcomers are advised to use the FORTRAN standard setup. After some practice with EGS4 tutorials given in Ref. /3/, it should be possible to understand the MORTRAN source of the standard user code and to tailor it for special purposes.

The analysis code of EGS4-HERMES generates four response data records at the end of each batch:

(1) EDEP@EOB (NREG,3,5)

energy deposition [MeV] per

- region 1 to NREG
- particle 1 e^-
 2 γ
 3 e^+
- type 1 deposition in flight
 2 deposition at ECUT/PCUT discard
 3 deposition at AE/AP discard
 4 leakage energy loss
 5 deposition of photo electrons generated
 below cutoff energy

(2) HARG@EOB (NREG,3,5)

frequency of analysis calls per

- region
- particle
- type see above

(3) PHIB@EOB (NREG,3)

flux by tracklength [cm] per

- region
- particle

(4) EESC@EOB (NREG,3)

leakage current [arbitrary units] per

- region
- particle

At end of run, the same information, averaged over all batches is reported using the following data record names:

EDEP@EOR	for energy depostion,	
HARG@EOR	for calling frequencies,	
PHIM@EOR	for tracklength flux, and	
EESC@EOR	for leakage current.	

3.4.1 EGS Input Description

EGS4 input/output is illustrated in Fig. 3.3.

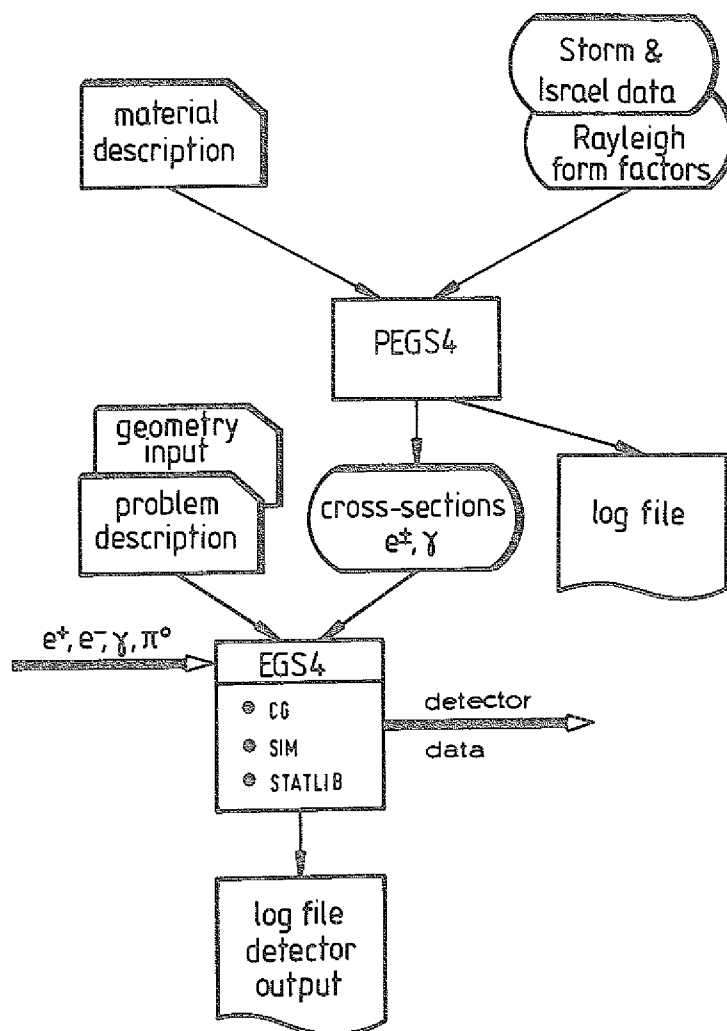


Fig. 3.3: EGS4/PEGS4 input-output

This input description refers to the standard EGS4 user code for the HERMES system. It is delivered along with some additional macros to enable the user to construct his own user code for special purposes.

Note, that a proper implementation of EGS4 into the HERMES system requires

- interfacing to combinatorial geometry (in HERMES version)!
- the correct coupling to HERMES submission files!

EGS4 requires upto four input files:

- Unit 5 contains the primary input (problem description) as described below.
- Unit INGEO is used for the geometry description. For details of HERMES-CG input refer to Section 3.6
- Unit KMPI is read by subroutine HATCH and must include the cross-section data generated by PEGS4.
- Unit INEGS can be used to read in an external particle source. This file must be written in HERMES submission file format as described in Section 2.5 of Chapter 2.

The primary input on unit 5 is organized in a single NAMELIST record &EGS. The items are described in Table 3.5.

Table 3.5

VARIABLE	Default	Description
IRUN	1	identification number of the run
MHIS	1	number of source particles started within one batch. For external particle sources MHIS is not used. If frequency distributions are to be measured, MHIS must be set to 1.
MBAT	10	maximum number of independent batches to be run.
INEGS	0	unit number of an external source particle file in HERMES submission file format. If INEGS=0 an internal particle source is used.
IOEGS	0	unit number of an output file in HERMES submission file format. This file is used to report the measured detector data for further processing in STATIST.
INGEO	5	input unit number for the combinatorial geometry input (refer to Section 3.6). If INGEO=5 CG input must follow the NAMELIST record.
IOGEO	6	output unit number for the combinatorial geometry print file (refer to Section 3.6)

KMPI	12	input unit number for the cross-sections prepared by PEGS4 (refer to Section 3.4.2 for PEGS4 input)
KMPO	8	output unit number for the cross-section print file generated by HATCH.
IXX	123456789	initial random seed. This should be a large odd integer number.
NMED		number of media used. NMED must not exceed 16 which is also the limit in HET. If more media are required the EGS4 user code is to be changed.
MEDNAM(NMED)		array of CHARACTER*24 strings containing the media names as used in PEGS4.
ESTEPE(NMED)		array of real numbers. ESTEPE limits the electron step size to an energy loss of ESTEPE(i) percent in one step for medium i. It is used by subroutine FIXTMX.
NREG		number of regions used in EGS4 (≤ 1000).
LSTREG(NREG)		array of integer numbers containing the region key for each region as assigned by the geometry package.
MED(NREG)		array of integer numbers containing the medium index of the media to be used for each region.
ECUT(NREG)		electron/positron cutoff energy for each region.
PCUT(NREG)		photon cutoff energy for each region.
IRAYLR(NREG)		integer switches (0=off, 1=on) to toggle coherent rayleigh scattering for each region.
IQI		particle type of source particles -1 - electrons 0 - photons +1 - positrons +2 - π^0

XI,YI,ZI	coordinates of source particles.
UI,VI,WI	direction cosines of source particles.
EI	total energy of source particles
WTI	statistical weight of source particles.

Note! IQI, XI,YI,ZI, UI,VI,WI, EI and WTI are not used for external particle sources (with INEGS>0)

3.4.2 PEGS4 Input Description

PEGS4 input/output is illustrated in Fig. 3.3.

PEGS4 must be used to generate cross section data for EGS4. The program is used as designed by Nelson et al. /3,4/ without any changes. Refer to /3/ for a detailed description of the code and input. Here we only give a compendium of the input description.

PEGS4 requires 3 input files:

- Input data for photoelectric, pair production and Rayleigh scattering (from Storm and Israel) is read from FORTRAN unit number 8.
- Input data for Rayleigh scattering form factors is read from FORTRAN unit 9.
- User input data, as described below, is read from unit 5.

PEGS4 generates output files:

- Data to be used by EGS4 is written on FORTRAN unit 7 in card image format.
- Primary print output is written on FORTRAN unit 6.
- A copy of the data provided for EGS is written on FORTRAN unit 10 in printer format.

PEGS user input consists of a set of processing options read in a loop. For each option PEGS4 reads

- a card containing the option name in (A4) format,
- the namelist record &INP containing the parameters used for the option,
- additional literal data needed for the option in fixed FORTRAN formats.

Cards A: Set Energy Cutoffs

A1:	FORMAT (A4)
	ENER

```

A2:          FORMAT (namelist)
             &INP
             AE = electron/positron cutoff energy [MeV]
             AP =  $\gamma$  cutoff energy [MeV]
             UE = highest electron/positron energy [MeV]
             UP = highest  $\gamma$  energy [MeV]
             &END

```

Cards A are followed by a medium specification:

Cards B: Setup an element medium

```

B1          FORMAT(A4)
             ELEM

B2          FORMAT (namelist)
             &INP
             RHO = density [g/cm3] (optional)
             WA  = atomic weight (optional)
             IRAYL = 1 to include Rayleigh scattering
                   = 0 otherwise
             IUNRST = 1 to allow unrestricted collision
                     stopping power
                   = 0 otherwise
             ISSB  = 1 to use user defined Sternheimer
                     density effect coefficients
                   = 0 otherwise
             AFACT = a (=0 for ISSB=0) (see Ref. 3)
             SK    =  $m_s$  (=0 for ISSB=0)
             X0    =  $x_0$  (=0 for ISSB=0)
             X1    =  $x_1$  (=0 for ISSB=0)
             CBAR  = C (=0 for ISSB=0)
             IEV   = I(eV) (=0 for ISSB=0)
             &END

B3:          FORMAT (A24,6X,A24)
             MEDIUM identifier assigned to medium data
             IDSTRN identifier of Sternheimer coefficients
                   predefined in PEGS (see Ref. 3)

B4:          FORMAT (A2)
             SYM atomic symbol of the element

```

Cards C: Setup a medium as a mixture of elements

```

C1:          FORMAT (A4)
             MIXT

```

```

C2:      FORMAT (namelist)
        &INP
        RHO = density of the mixture [g/cm3]
        NE  = number of elements in mixture
        RHOZ(I) = relative amount of element I in
                  mixture (must be given for all
                  NE elements)
        GASP      = 0. for solid or liquid media
                  (default)
                  = gas pressure [atm.]
        WA(I)     = atomic weight of element I
                  (optional)
        IRAYL     = 1 to include Rayleigh scattering
        IUNRST    = 1 to allow unrestricted collision
                  stopping power
        ISSB      = 1 to use user defined Sternheimer
                  density effect coefficients
        AFACT     = a  (=0 for ISSB=0) (see Ref. 3)
        SK        = ms (=0 for ISSB=0)
        XO        = x0 (=0 for ISSB=0)
        X1        = x1 (=0 for ISSB=0)
        CBAR      = C  (=0 for ISSB=0)
        IEV       = I(eV) (=0 for ISSB=0)
        &END

```

```
C3:      FORMAT (A24,6X,A24)
        MEDIUM identifier assigned to media data
        IDSTRN identifier of Sternheimer coefficients
              predefined in PEGS (see Ref. 3)
```

```
C4:      FORMAT (24(A2,1X))
        SYM(I) atomic symbols of all NE elements
```

Cards D: Setup a medium as a compound of NE elements

```
D1:                FORMAT (A4)
                   COMP
```

```

D2:          FORMAT (namelist)
            &INP
            RHO = density of the compound [g/cm3]
            NE  = number of elements in compound
            PZ(I)    = relative number of atoms of
                        element I in compound
                        (must be given for all NE elements)
            GASP     = 0 for solid or liquid media
                        (default)
                        = gas pressure [atm.]

```

WA(I) = atomic weight of element I
 (optional)
 IRAYL = 1 to include Rayleigh scattering
 = 0 otherwise
 IUNRST = 1 to allow unrestricted collision
 stopping power
 = 0 otherwise (default)
 ISSB = 1 to use user defined Sternheimer
 density effect coefficients
 = 0 otherwise (default)
 AFACT = a (=0 for ISSB=0) (see Ref. 3)
 SK = ms (=0 for ISSB=0)
 XO = x_0 (=0 for ISSB=0)
 X1 = x_1 (=0 for ISSB=0)
 CBAR = C (=0 for ISSB=0)
 IEV = I(eV) (=0 for ISSB=0)
 &END

D3: FORMAT (A24,6X,A24)
 MEDIUM identifier assigned to media data
 IDSTRN identifier of Sternheimer coefficients
 predefined in PEGS (see Ref. 3)

D4: FORMAT (24(A2,1X))
 SYM(I) atomic symbols of all NE elements

After each medium definition the "piecewise linear fit" option must be called to compute the cross section set needed by EGS.

Cards E: invoke the "piecewise linear fit"

E1: FORMAT (A4)
 PWLF

E2: FORMAT (namelist)
 &INP &END
 normally no parameters are needed. For
 details refer to Ref. 3.

When the cross sections have been computed with PWLF use the DECK option to store them on FORTRAN unit 7 for further processing in EGS.

Cards F: save cross section data for EGS

F1: FORMAT (A4)
 DECK

F2: FORMAT (namelist)
 &INP &END
 no parameter needed

For subsequent media the input must be continued with another medium setup (ELEM, MIXT or COMP) followed by PWLF and DECK. The last option must be the STOP option:

Cards G: terminate the PEGS run

G1: FORMAT (A4)
 STOP

G2: FORMAT (namelist)
 &INP &END
 no parameters needed

Note! A slightly modified user code has been developed to run EGS4 on the CRAY-XMP computer.

3.5 NDEM Input Description

NDEM input is illustrated in Fig. 3.4.

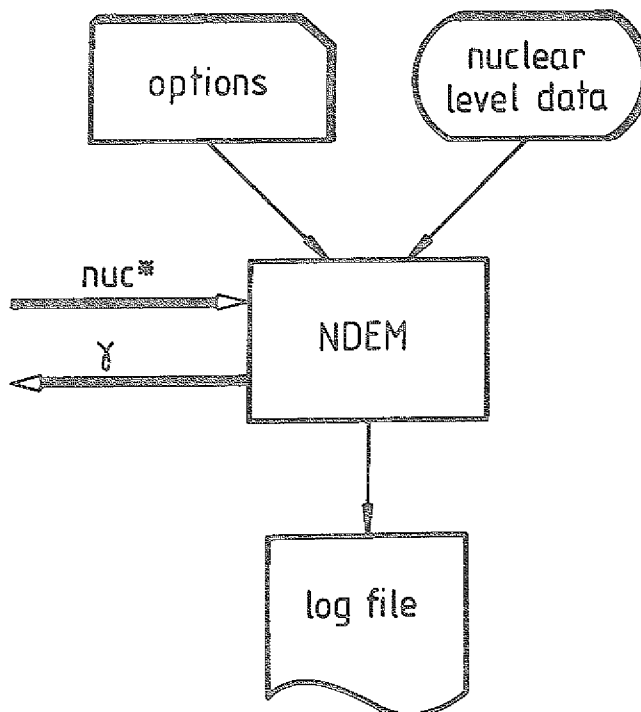


Fig. 3.4: NDEM input-output

The NDEM run consists of two steps:

- TRX: translation of the level library from ASCII to binary code as read in NDEM
- NDEM: calculation of the gamma rays from residual nuclei on HERMES submission file written by HET

(1) Run of TRX

TRX requires two input units:

- Unit 12 is used for primary input as described below
- Unit INBCD is used for BCDLIB, the level library in ASCII code

TRX requires three output units:

- Unit 6 is used for primary output
- Unit IOPHT is used for PHTLIB, the level library in binary code
- Unit LOUT is used for printed output (optional)

TRX input is written in card image format as described below

Card TRX1	FORMAT (*)
INBCD	Logical unit number for level library ASCII file
IOPHT	Logical unit number for level library binary file
LOUT	Logical unit number to print level data, if > 0

Note! The file LOUT has more than 15000 lines.

Print is only usefull for general information about level library.

(2) Run of NDEM

NDEM requires three input units

- Unit 5 is used for primary input as described below
- Unit INUCTP is used for HERMES submission file containing the residual nuclei data
- Unit 30 is used for level library PHTLIB as created by TRX in the first step

NDEM requires two output units

- Unit 6 is used for primary output
- Unit IGAMTP is used for NDEM output file containing generated γ rays in HERMES submission file structure

NDEM input is written in card image or REA format as described below and in Section 3.3.1.

Card PHT1	FORMAT(*)
ICC	The level of physics to be applied. Values of 0-4. (For detailed information see Section 3.5.1)
NERG	The number of energy bin boundaries to represent the de-excitation γ spectra, if >0; if <0, [NERG] energy bin boundaries from 10 keV in steps of 10 keV will be calculated
FNORM	An overall weighting factor to be applied to the printer and plotter output. The default is 1.
KPLOT	flag to turn on plotting. =0 implies no plotting, default. =1 requests plots
Card PHT2	Format(*)
INUCTP	unit number of HERMES submission file containing residual nuclei
IGAMTP	unit number of HERMES submission file to hold generated γ rays
ICPL	switch to copy neutron records from INUCTP to IGAMTP, if > 0 ; usefull if IGAMTP should be input tape for MORSE
Card PHT3	FORMAT(REAG)
(omit, if NERG <=0)	
ERGB	NERG upper energy bin boundaries (in MeV) for editing the de-excitation γ spectrum is required. NERG energies may be entered from low to high.

3.5.1 The NDEM Model Option ICC

(1) ICC=0 : The Pure Continuum

When ICC=0 is specified, the continuum is assumed to exist at all nuclear excitations above the pairing gap for the residual nucleus. As a consequence, there is no line spectrum but rather only a continuous γ energy distribution. All transitions, within the continuum or from the continuum to the ground state, are assumed to be E1 transitions.

(2) ICC=1 : The Troubetzkoy (E1) Model

For ICC=1, the nuclear energy levels specified in the data library are used, the continuum is assumed to begin above the highest specified level. However, all transitions, whether continuum-to-continuum, continuum-to-level or level-to-level, are assumed to be E1 transitions.

(3) ICC=2 : The Intermediate Model

When ICC=2, the model employed is really a hybrid between models (2) and (4). The nuclear levels and their spins and parities are obtained from the data library. The continuum begins above the highest specified level. However, all levels above the lowest that has unknown spin and parity are also considered to have unknown spin. All transitions within or from the continuum, or that involve a level with unknown spin, are treated as E1 transitions as in model (2). All transitions involving only states of known spin and parity utilize the Weisskopf single-particle transition model as in option (4).

(4) ICC=3 : The Spin Dependent Model

When ICC=3 option gives the full model as described as option (1), with the exception that any experimental branching ratios are ignored and a pure model calculation is performed for the transition probabilities using the Weisskopf single-particle transition estimates.

(5) ICC=4 : The Full Model with Experimental Branching Ratios

With ICC=4, the full procedure described as option (1) is employed, including the use of library-specified branching ratios when available.

3.6 Description of Combinatorial Geometry Input

Combinatorial geometry input can be written either in card image format, as described in the Section 3.6.2, or in CMD command language. In CMD the user need not care about column formats, he can use symbolic variables instead of constants to define geometry data, and he can use general CMD commands like LOOP etc. to simplify the input for regular geometric patterns.

3.6.1 Combinatorial Geometry Input in CMD Language

The user creates a file containing the geometry description in CMD language. Then CMD must be called to compile this file to card-image format input, which can be included in HET, EGS or MORSE input files.

This should be done preferently using a suitabel procedure within an interactiv operating system. For IBM-CMS users the REXX procedure GEOGEN is available on the HERMES distribution tape.

A summary of CG related CMD commands is given below.

(1) Use GEOCMP Command to Start the Geometry Description.

```
GEOCMP nbody nzone nczone lenfpd lenma;
      nbody = number of bodies used
      nzone = number of input zones used
      nczone= number of code zones used
      lenfpd= length of floating point storage needed
      lenma = length of integer storage needed
```

(2) Use the Following Commands to Define Bodies.

ARB	k vv1 ... vv8 is1 ... is6 ;	arbitrary polyhedron
SPH	k vv r;	sphere
RCC	k vv hh r;	rectangular circular cylinder
REC	k vv hh rr1 rr2;	" elliptic cylinder
TRC	k vv hh r1 r2;	truncated rectangular cone
ELL	k vv1 vv2 b;	ellipsoid
BOX	k vv hh1 hh2 hh3;	box
WED	k vv hh1 hh2 hh3;	wedge
RPP	k xmin xmax ymin ymax zmin zmax;	rectangular parallel epiped
P	k uu d ;	plane (hesse form)
PX	k d;	plane rectangular on x (y,z plane)
PY	k d;	plane rectangular on y (x,z plane)
PZ	k d;	plane rectangular on z (x,y plane)
PS	k vv uu d;	plane (hesse form,shifted)
C	k vv uu r;	cylinder
CX	k vv r;	" uu = x direction
CY	k vv r;	" uu = y direction
CZ	k vv r;	" uu = z direction
END	;	end of body input

k = body key number
 vv = vertex vector
 uu = direction cosinus of axis
 rr = radius vector
 hh = hight vector
 r = scalar radius
 d = scalar distance used in Hesse-form

(3) Use the Following Commands to Define Zones

ZONE imed ireg condition-list; define geometry zone
 SUBGEO ; start subgeometry-zone set

define the medium returned by the geometry for the zone
 imed = -1 for external (black) voids
 = 1000 for internal voids in MORSE
 = 6666 for internal voids in HET and EGS
 < -1 to leave medium key undefined

define the region number returned by CG for the zone
 ireg > 0 to define region number
 < 0 to define only 3 digits of a composed 9 digit
 region number

define the geometrical domain of the zone-
 condition list
 syntax of condition list:
 code-zone (OR code-zone (...)) (GO set-number)

syntax of code-zone:
 k1 k2
 ki>0 means all code-zone points lie inside body ki
 ki<0 means all code-zone points lie outside body ki

OR operator may be used to join code-zones together

GO may be used to pass control to a subgeometry-zone
 set to subdivide the zone into subzones. The top
 set of zone descriptions refers to set number 1.
 When SUBGEO is encountered the set number is incre-
 mented

In set number 1 ireg<0 will generate nreg=-ireg*100000

The set called by GO from set number 1 will generate
 nreg = ireg if ireg>0 or
 nreg = nreg -ireg*1000 if ireg<0

The set called from this level will generate
 $nreg = ireg$ if $ireg > 0$ or
 $nreg = nreg - ireg$ if $ireg < 0$

Thus, the maximum nesting level of zone descriptions is 3,
 and the minimum ireg per level is -999.

(4) Terminate Card Image Input by ENDGEO

ENDGEO ;
 terminates the geometry input, the object output is
 now available on unit 22.

(5) Use the Following Commands to Test the Geo Description

GEOIN in,io; load geo object
 invokes subroutine JOMIN of the CG-L2 package
 to read card image input.

LOOKZ ss; query point properties
 RAY ss uu raylength; ray tracing
 PICM ss uu vv size; medium picture
 PICR ss uu vv size; region picture
 PICZ ss uu vv size; zone picture
 NEXT ; prepare for next picture
 MARK iarea kline; shade "area" with "kline"
 HEAD no text; define label 1 TO 7

ss shift vector
 uu direction cosines of u axis
 vv direction cosines of v axis

3.6.2 Combinatorial Geometry Input in Card Image Format

Card A: FORMAT(*) Header card 1

NUMB number of bodies (default 100)
 NUMR number of input zones (default 100)
 NCOD number of code zones (default 100)
 LFPD length of floating point data needed (default 1000)
 LMA length of integer data needed (default 1000)
 LOUT not used
 IDBGD if 0, zone overlap will be checked by LOOKZ

Card B: FORMAT(*) Header card 2

IVOPT volume option
 0 set all volumes equal to 1.0
 1 invalid in this version
 2 invalid in this version
 3 read in NUMR volumes
 4 bypass volume calculation

MEDREG boundary option
 -1 return from GEOM at region boundaries
 0 return from GEOM at all boundaries
 1 return from GEOM at medium boundaries

Card C: FORMAT(*) Header card 3

IPRINT taylor CG printing
 0 no print
 1 print geometry description
 2 print description and storage dump

STRETCH scale factor (default=1.)

Cards D: FORMAT(2X,A3,1X,I4,6E10.3) Body cards

TYPE Body type
 INR body number
 PARAM₁ up to six parameters describing the body

Note! There must be a card D for each of the NUMB bodies.
 The last one must be "END".
 If a body requires more than six parameters, continuation
 cards must be used leaving TYPE and INR blank.
 See Table 3.6 to find the parameters needed for body
 input.

Table 3.6
CG-Body Types and Parameters

ARB	k vv1...vv8 is1...is6 ;	arbitrary polyhedron
SPH	k vv r;	sphere
RCC	k vv hh r;	rectangular circular cylinder
REC	k vv hh rr1 rr2;	" elliptic cylinder
TRC	k vv hh r1 r2;	truncated rectangular cone
ELL	k vv1 vv2 b;	ellipsoid
BOX	k vv hh1 hh2 hh3;	box
WED	k vv hh1 hh2 hh3;	wedge
RPP	k xmin xmax ymin ymin zmin zmax;	rectangular parallel epiped
P	k uu d ;	plane (hesse form)
PX	k d;	plane rectangular on x (y,z plane)
PY	k d;	plane rectangular on y (x,z plane)
PZ	k d;	plane rectangular on z (x,y plane)
PS	k vv uu d;	plane (hesse form,shifted)
C	k vv uu r;	cylinder
CX	k vv r;	" uu = x direction
CY	k vv r;	" uu = y direction
CZ	k vv r;	" uu = z direction
END	;	end of body input

k body key number
 vv vertex vector
 uu direction cosines of axis
 rr radius vector
 hh hight vector
 r scalar radius
 d scalar distance used in Hesse-form

Cards E:FORMAT(2I5,9(A2,I5)) Input zone cards

IMED medium number
 use 1000 for internal void
 -1 for external void
 -2 to leave unspecified
 777 to indicate start of subgeometry
 999 to terminate zone input
 blank on continuation lines

IREG region number (blank on continuation cards)
 use IREG>0 for normal regions
 blank on continuation lines
 IREG<0 to define three-digit part of the region
 number in connection with the GO operator

OPI,Ni up to 9 geometry conditions

Note! An input-zone consists of one or more code-zones.

Each code zone "CZ" describes a set of points in space:
 [(Xcz,Ycz,Zcz)]

Code zones may be combined to one input-zone using the
 OR operator on card E.

A code zone is defined by one or more bodies, referenced
 by number "Ni".

If Ni>0, all points of the code-zone must lie inside body
 Ni; if Ni<0, all points of the code-zone must lie outside
 body Ni.

If OPI="GO" , then Ni refers to the input-zone card set
 which is called for subdividing the zone into subzones.
 Input-zone card sets are numbered sequentially. On input
 they are separated by a card with IMED=777.

Zone input is terminated by a card with IMED=999.

3.7 SIM Input Description

Detector input must be coded prior to the primary HET input,
 using CMD commands. The analysis input consist of:

- One DEFDET command for each detector wanted
- A SIMIO command to specify the binary output of the detectors
- A RUN command to switch back to the HET input.

At the end of run (i.e., after the HET input) additional
 commands can be used to normalize detector results etc. This
 input section must be terminated with the EXIT command.

There are some additional CMD commands which can be used in both
 analysis input sections to assist the user in defining data for
 binning, normalization and some other useful techniques, e.g. to
 enter data into arrays. These commands are described in the CMD
 command language summary given in Section 3.9.

Here we will describe an easy to use method only:

3.7.1 CMD Commands for SIM Input

DEFDET name, type, prt, ud, ebin, abin, IR1, IR2;

- name - a four-character name used to identify all arrays associated with the detector
- type - the integer representation of the detector type as described in Section 3.7.2.
- prt - a real value indicating the particle type to be taken into account (not used for detector types 4, 5, and 9)
- | | | | |
|-------------|--------------|----|------------------------|
| use prt = 0 | for protons | 7 | for deuterons |
| 1 | for neutrons | 8 | for tritons |
| 2 | for π^+ | 9 | for ^3He ions |
| 3 | for π^0 | 10 | for α 's |
| 4 | for π^- | | |
| 5 | for μ^+ | | |
| 6 | for μ^- | | |
- for flux and current detectors also prt=-1 can be used to score source particles only.
- ud - is a cartesian unit vector, indicating a reference direction of the detector used for angle binning. It may be specified by three real constants.
- ebin - is a list of energy-bin boundaries in [MeV] ordered from high to low (used in detectors 1-3 and 6-8, if desired).
ebin may be coded as a list of real constants enclosed in < >, all values separated by blanks.
- abin - is a list of angles [$^\circ$] ordered from low to high for the angular binning in detectors of types 1-3 and 6-8. Abin may be coded as a list of real constants enclosed in < >, all values separated by blanks.
- IR1 - is a list of region numbers as defined by the geometry defining the regions to be used with the detector.
IR1 may be coded as a list of integer constants enclosed in < > all values separated by blanks.

IR2 - is a second list of region numbers used to define surfaces.
 For detector types 6, 7 and 9 a surface is identified by the common surface of two regions IR1(i)

Parameters of the DEFDET command, which are not used, may be omitted coding position holding commas only.

SIMIO [IOSIM iosim] [IORSY iorsy];

iosim - unit number of a binary file on which the detector results are to be stored after each batch and at the end-of-run in HERMES submission file structure.
 If iosim is specified, data records are generated for each detector defined. The record names are constructed of the four-character detector name followed by the string "@EOB" at the end of batch and "@EOR" at the end-of-run.

iorsy - unit number of an RSYST file /1/ including the end of run results of the detectors.

RUN;

The RUN command terminates the first analysis-input section and returns control to HET-input reading from the primary input.

EXIT;

EXIT immediately stops the execution of the program

3.7.2 Available Estimator Types

(1) Secondary Particle Yield Detectors

DEFDET name,1,prt,[ud],[ebin],[abin],[ir1];
 particles generated in INC
 prt may be 0,...,6. for p,n, π^+ , π^0 , π^- , μ^+ , μ^-

DEFDET name,2,prt,[ud],[ebin],[abin],[ir1];
 particles generated in evaporation and fission
 prt may be 0.,1.,and 7,...10. for p,n,d,t,He³, α

```
DEFDET name,3,prt,[ud],[ebin],[abin],[ir1];
particles generated in INC, evaporation and fission.
prt may be 0,...,10.
```

When we define:

```
E - secondary particle energy
 $\omega$  - the angle between the secondary particle flight
direction and the detector reference direction "ud"
 $\theta$  - the angle between the secondary particle flight
direction and the direction of the projectile
particle (ud must not be given)
IR - the region where the collision occurred.
```

then we can setup histograms of the following functions using the given DEFDET commands

```
Y DEFDET name,type,prt;
Y(E) DEFDET name,type,prt,,ebin;
Y( $\omega$ ) DEFDET name,type,prt,ud,,abin;
Y( $\theta$ ) DEFDET name,type,prt,,,abin;
Y(IR) DEFDET name,type,prt,,,,ir1;
Y(E, $\omega$ ) DEFDET name,type,prt,ud,ebin,abin;
Y(E, $\theta$ ) DEFDET name,type,prt,,ebin,abin;
Y(E,IR) DEFDET name,type,prt,ebin,,ir1;
Y(E, $\omega$ ,IR) DEFDET name,type,prt,ud,ebin,abin,ir1;
Y(E, $\theta$ ,IR) DEFDET name,type,prt,,ebin,abin,ir1;
Y( $\omega$ ,IR) DEFDET name,type,prt,ud,,,abin,ir1;
Y( $\theta$ ,IR) DEFDET name,type,prt,,,abin,ir1;
```

Example 1: Measurement of the number of protons generated by evaporation and fission in regions 1 and 20 as function of energy Y(E,IR)

```
REAL EB(11);
DATA EB=<100. 90. 80. 70. 60. 50. 40. 30. 20. 20. 0.>;
DEFDET PEVA,2,0.,,EB,,<1 20>;
```

This example also illustrates the use of auxiliary CMD commands, which help in defining data referencing them by name. The DATA statement could also be coded as

```
DATA EB=<100. A 10 -10.>;
```

using fast-array input format. Refer to CMD language summary for details. (Section 3.9)

Example 2: Measurement of the angular distribution of all secondary neutrons with respect to the z direction in steps of 10 degrees.

```
DEFDET NANG,3,1.,0. 0 .1.,,<0. A 9 10>;
```

again the binning data is coded in fast-array input format.

(2) Energy Deposition Detector

```
DEFDET name,4,,,,[ir1];
```

energy and angle binning is not allowed for detectors of type 4.
"prt" must not be specified.
ir1 may be used to specify the regions where the detector is to measure the energy deposition.

(3) Scintillation Light Detector

```
DEFDET name,5,,,,[ir1];
```

energy and angle binning is not allowed;
"prt" must not be given
"ir1" may be used to specify the regions where scintillation light is to be measured.

omitting "ir1" in detector types 4 and 5 results in a summary over the whole geometry

```
DEFDET name,4;
```

```
DEFDET name,5;
```

(4) Forward Current Detectors

Count of the number of particles of type "prt" passing through the region boundaries denoted by "ir1" and "ir2", optionally binned in energy and angle per source particle.
Particle types allowed are 0.,1.,2.,4.,5.,6. for p, n, π^+, π^- , μ^+, μ^- and -1. for source particles

```
DEFDET name,6,prt,[ud],[ebin],[abin],ir1;
```

count particles leaving the regions ir1(i)

```
DEFDET name,6,prt,[ud],[ebin],[abin],,ir2;
```

count particles entering the regions ir2(i)

```
DEFDET name,6,prt,[ud],[ebin],[abin],ir1,ir2;
```

count particles passing from ir1(i) to ir2(i)

Consider

IS as the surface index according to ir1 and ir2.
 E as the particle energy on boundary
 ω as the angle between the particle flight direction
 and the detector reference direction "ud"
 ψ as the angle between the particle flight direction
 and the normal vector of the surface at the inter-
 section point,

then we can setup histograms of the forward current J^+ using
 the following DEFDET commands.

```
J+(IS)      DEFDET name,6,prt,,,[ir1],[ir2];
J+(E,IS)    DEFDET name,6,prt,,ebin,,[ir1],[ir2];
J+( $\omega$ ,IS)   DEFDET name,6,prt,ud,,abin,[ir1],[ir2];
J+( $\psi$ ,IS)    DEFDET name,6,prt,,abin,[ir1],[ir2];
J+(E, $\omega$ ,IS)  DEFDET name,6,prt,ud,ebin,abin,[ir1],[ir2];
J+(E, $\psi$ ,IS)  DEFDET name,6,prt,,ebin,abin,[ir1],[ir2];
```

Example 1: Count all μ^- escaping from the geometry.
 The external void has region number 99.
 DEFDET MUES,6,6,,,,,<99>;

Example 2: Compute a double differential spectrum of n
 entering regions 1 and 2 with respect to the
 z direction in 1 MeV energy bins and 30° angle
 bins.
 DEFDET NUES,6,1.,0. 0. 1.,
 <1000. A 999 -1.>
 <0. A 7 30.>,,<1 2>;

(5) Forward Surface Flux Detector

Measurement of the flux of particles of type "prt" passing
 through the common region boundaries denoted by "ir1" and "ir2"
 per source particle,
 optional energy and angle binning is provided. Particle types
 allowed are 0.,1.,2.,4.,5.,6. for p,n, π^+ , π^- , μ^+ , μ^- and -1. for
 source particles.

```
DEFDET name,7,prt,[ud],[ebin],[abin],[ir1],[ir2];
```

for details of the command syntax refer to the description
 of the forward current detector.

(6) Tracklength Flux Detectors

Measure the (volume integrated) flux of particles of type "prt" in the regions specified by "ir1". Optional energy and angle binning is provided. Particle types allowed are 0.,1.,2.,4.,5.,6. for p,n, π^+ , π^- , μ^+ , μ^- and -1. for source particles.

```
DEFDET name,8,prt,[ud],[ebin],[abin],[ir1];
```

When ω is the angle between the particle-flight direction and the detector-reference direction "ud" and E is the particle energy and IR is the considered region then histograms of the flux ϕ [cm] can be defined as:

```
 $\phi$           DEFDET name,8,prt;
 $\phi(E)$       DEFDET name,8,prt,,ebin;
 $\phi(\omega)$    DEFDET name,8,prt,ud,,abin;
 $\phi(IR)$       DEFDET name,8,prt,,,ir1;
 $\phi(E,\omega)$   DEFDET name,8,prt,ud,ebin,abin;
 $\phi(E,IR)$     DEFDET name,8,prt,,ebin,,ir1;
 $\phi(\omega,IR)$  DEFDET name,8,prt,ud,,abin,ir1;
 $\phi(E,\omega,IR)$  DEFDET name,8,prt,ud,ebin,abin,ir1;
```

(7) Detector for Particle Energy Passing Given Surfaces

This detector counts for the energy of all particles passing through a given surface. All particle types are considered simultaneously. "prt" need not be specified. Energy and angle binning is not provided.

```
DEFDET name,9,,,,,ir1;
      counts particle energy of particles having
      region ir1(i)
DEFDET name,9,,,,,ir2;
      counts particle energy of particles entering
      region ir2(i)
DEFDET name,9,,,,,ir1,ir2
      counts particle energy of particles passing
      from ir1(i) to ir2(i)
```

When IS is the surface index as requested by ir1(i) and ir2(i) a table of energy E^+ is generated

```
 $E^+(ptyp,IS)$ 
      with ptyp is p n  $\pi^+$   $\pi^-$   $\mu^+$   $\mu^-$ 
```

3.8 STATIST Input Description

STATIST input/output is illustrated in Fig. 3.5.

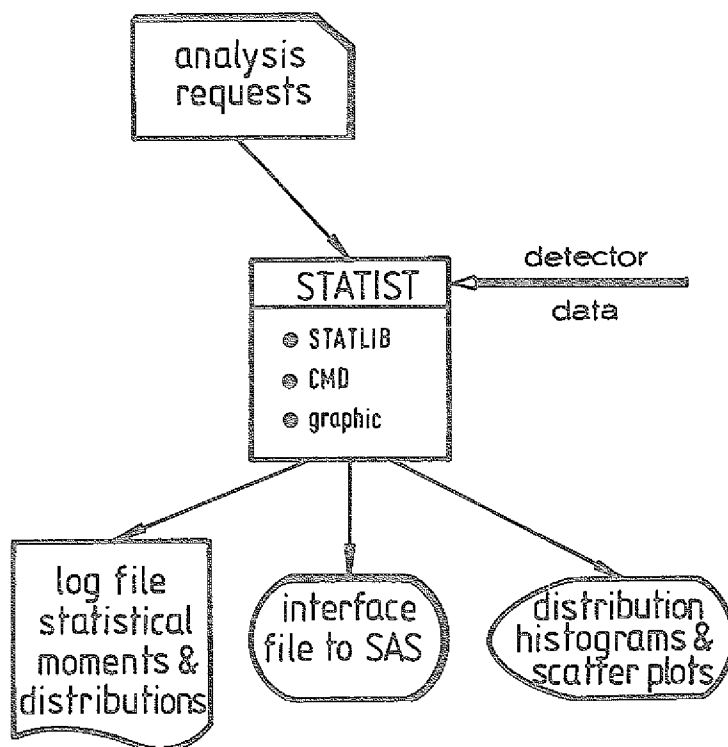


Fig. 3.5: STATIST input-output

STATIST input consists of

- one or more HERMES submission files containing detector output data from Monte Carlo runs,
- a command file containing the user requests of how to select, combine and present the data stored on the submission files. The command file is read from unit 5 and must be coded in CMD command language (see Section 3.9).

3.8.1 Structure of the STATIST Command File

(1) Initialisation Section

The user must allocate array variables to keep data that are loaded from submission files, data computed in the data entry section or static arrays like weighting tables etc.

(2) Data Entry Section

This section represents a loop over all events (batches).

It comprises commands to

- load data from submission files,
- for arithmetical and logical operations on loaded data, resulting in a set of variables V_i of each event on a file.

(3) Presentation (Analysis) Section

In this section the user can request histograms of

- the frequency distribution functions of V_i ,
- the statistical moments of V_i ,
- analysis of correlations between variables.

3.8.2 Syntax of STATIST Commands

The commands used in STATIST are CMD commands; for a complete description of all CMD commands refer to the CMD command language summary (Section 3.9).

(1) Initialization Section

Allocation of arrays:

```
REAL name (n1,n2,n3) [...];
```

```
INTEGER name (n1,n2,n3) [...];
```

Array specification is done like in FORTRAN, upto 3 dimensions are allowed.

Initialization of arrays:

```
DATA name = <list of data elements>;
```

only one array can be initialized in a DATA command. Subsequent DATA commands for the same array name overwrite the preceeding assignment.

The list of data elements may include constants and symbolic sequences of constants (like Rnv for repeat constant v n times or Anv for add n-times constant v).

(2) Data Entry Section - Event Loop

This section begins with the STATIN command:

```
STATIN nv,nevt;
```

STATIN establishes a buffer for "nv" variables and "nevt" events.

LOAD unit, record name, array;
 scans the HERMES submission file associated with unit number "unit" for the next occurrence of a data record named "record name" and reads this record into "array".

SET dest = srs₁ [srs₂][+...];d

dest may be a variable name of dimension 1,
 a subscripted array name.
 srs_i may be a variable of dimension 1,
 a subscripted array name,
 a constant.

SET can be used to compute linear combinations of loaded data items or to apply weights.

DOT d = s₁*s₂ [(n₁,n₂,n₃)];

d array name of the destination matrix

s_i array names of the source matrices

(n₁,n₂,n₃) if coded, then the dimensions of the arrays d,

s₁ and s₂ are temporarily overwritten and

d(n₁,n₃)=s₁(n₁,n₂)*s₂(n₂,n₃) is calculated.

DOT calculates the inner product (DOT product) of matrix s₁ and matrix s₂ and stores it into matrix d.

DOT can be used to fold results with weighting tables.

Logical operations:

IF condition;

AND condition;

OR condition;

condition may be srs₁=srs₂
 srs₁<srs₂
 srs₂>srs₂

IF, AND, and OR set the condition code examined in the commands LIST and DROP.

DROP;

causes the current event to be discarded from the analysis.

DROP is executed only if the condition code set by IF, AND, or OR is true.

LIST;

if the condition code set by IF, AND or OR is true, the current event variables V_i are printed.

VETO condition;

condition may be variable < constant
 variable = constant
 variable > constant

VETO can be used to discard an event from analysis.

VETO is equivalent to IF condition;DROP;

(3) Presentation (Analysis) Section

```

    BIN iv, from, to, nbin, label;
    iv      - the variable index of the variable V to be used
              in construction of a histogram
    from    - lowest binning limit
    to      - highest binning limit
    nbin    - number of binning intervals to be used
    label   - descriptive text to be used in histogram
              output enclosed in inverse commas.

```

The BIN command results in the construction of a frequency distribution plot of $V(iv)$, the computation of the empirical moments of $V(iv)$ and the grouped moments of the associated histogram.

```

    FIT iv, from, to;
    iv      - index of the variable V(iv) to be fitted
    from    - lowest value of V(iv) to be used in the fit.
    to      - highest value of V(iv) to be used in the fit.
    FIT computes the parameters of a Gaussian fit of the
    frequency distribution function of V(iv) in the range
    <V(iv)<to.

```

```

    SCAT ivx, ivy;
    ivx     - index of V(ivx) to be used on x axis
    ivy     - index of V(ivy) to be used on y axis.
    SCAT generates a scatter plot of V(ivy) vs. V(ivx) in the
    value window as specified in the corresponding BIN commands.
    Scatter plots can be used to examine the correlation between
    variables. The correlation factor is computed automatically.

```

This section ends with the STATIST command.

```

    STATIST ioplot, splot, IOSAS;
    ioplot may be "print" to select line printer plotting or
    "plot" to select plotter software, which has to be
    user provided.
    splot is the size of viewgraphs in cm, if plotter
    software is chosen. The features of this software are
    discussed in Appendix D.
    IOSAS is a unit number to store V for analysis in SAS.
    "STATIST" must be followed by "nv" BIN commands to
    specify the histogram parameters for the variable V(iv)
    with iv=1 to nv.

```

The STATIST input must be terminated by the EXIT command.

```

    EXIT;

```

3.9 CMD Command Language Summary

3.9.1 CMD Syntactic Rules

In this chapter we describe the syntactic rules for all input parts entered in CMD command language.

CMD commands begin with a keyword (the command verb) identifying the function of the command, followed by a list of parameters and a semicolon (;).

verb parameter-list;

Parameters can be:

- integer constants 123, -7
- real constants -4.5, 3.12E-10
- integer array constants <2 9 15>
- real array constants <1. 3. 7. 10.>
- character strings 'string'
- vector constants \$K 10. 1. 7.
- variable names A, RVAL, I
- subscripted variable names B(1,I,J)

- Integer, real and character constants can be entered as in FORTRAN read statements.
- Vector constants are used to enter the coordinates of points in space or directions in space. They can be entered 5 ways:
 - 1) x y z in kartesian coordinates
 - 2) \$K x y z in kartesian coordinates
 - 3) \$S r ϕ θ in spherical coordinates
 - 4) \$C r ϕ θ in cylindrical coordinates
 - 5) \$N symbolic for \$K 0. 0. 0.
 - \$X symbolic for \$K 1. 0. 0.
 - \$Y symbolic for \$K 0. 1. 0.
 - \$Z symbolic for \$K 0. 0. 1.

x, y, z, r, ϕ , θ may be real constants, names of real variables and subscripted names of real array variables.

- Array constants are a list of constants and series expansions separated by blank and enclosed in < >.
- The following syntax items are allowed in array constant input:

- c - constant
- R n c - repeat n-times c
- Q n m - repeat n-times the last m values
- A n c - add n-times c to the last c
- M n c - multiply n-times with c
- L c₁ c₂ c₃ - generate c's starting with c₁ to c₃ in steps of c₂

c_1 I n c_2 - interpolation of n values between
 c_1 and c_2 in n steps
 F c - fill array with c

3.9.2 Commands Used to Define User Variables

REAL name (n_1, n_2, n_3) [...];
 INTEGER name (n_1, n_2, n_3) [...];

name - 1 to 8 characters first alphabetic
 n_i - integer constant or variable specifying up to
 3 dimensions

Data can be entered into user variables by the DATA command
 or the assignment statement.

DATA name = <array constant>;

array constants are explained above
 Assignment statement;

destination=expression;

where destination may be the name of a single variable or a sub-
 scripted array name,

and expression may consist of constant-operands, variable or
 subscripted array variables connected by numerical opera-
 tors like in FORTRAN. Some numerical functions can be
 used:

SIN, COS, TAN	sin, cos and tan of x
SIND, COSD, TAND	sin, cos and tan of angle in degrees
ARC, DEG	convert degrees to x and vice versa
EXP, LOG, LOG10	exponential and logarithmic
SQRT	square root
CEIL, FLOOR	next integer >x or <x, respectively

some functions are used in a different way than in FORTRAN
 conventions:

MIN(array)	minimum of all elements of an array
MAX(array)	maximum of all elements of an array
PMIN(array)	minimum of all positive elements of an array
PMAX(array)	maximum of all positive elements of an array
SUM(array)	sum of all elements of an array.

When the destination of an assignment statement is an array or a subarray written as

```
name(l1:h1, l2:h2, l3:h3),
```

then the assignment expression is evaluated for each subscript of the destination in a loop structure

```
DO i3=l3,h3
DO i2=l2,h2
DO i1=l1,h1
name (i1,i2,i3) = expression (i1,i2,i3)
ENDDO
```

In array assignments the expression may include arrays and subarrays. The subscripts of array operands are incremented automatically for each evaluation of the expression. When there are constant arguments, undimensioned variable operands or array operands arguments with fixed subscript in the expression their associated values are repeated for each evaluation of the expression. If the automatic incrementation of the subscript of an array operand runs out of the subscript range, the last valid subscript is used for all subsequent evaluations of the expression. See the following examples for illustration:

Example 1

```
A(3:5,1) = B(7:9) + C*3.;
results in
A(3,1) = B(7) + C*3.;
A(4,1) = B(8) + C*3.;
A(5,1) = B(9) + C*3.;
```

Example 2

```
REAL A(5), B(2);
A = B*5.;
results in
A(1) = B(1)*5.;
A(2) = B(2)*5.;
A(3) = B(2)*5.;
A(4) = B(2)*5.;
A(5) = B(2)*5.;
```

3.9.3 Commands Used to Control User Variables

QN;

Query Names displays a list of all user variables allocated.

DN

name;

Display Name displays the values associated with a user variable

SAVE name;
 writes a user variable on unit 7

3.9.4 Commands Used to Control the Program Flow of the Input Interpreter

EXIT;
 terminates the program.

RUN;
 return to the calling application program.

UNIT iunit;
 continue input interpretation on unit iunit.

LOOP n;
ENDL;
 the input between LOOP and ENDL is repeated n-times.

FOR ivar=i1,i2,i3;
ENDFOR;
 the input between FOR and ENDFOR is repeated like a
 FORTRAN DO-loop.
ivar name of an integer variable
i1 integer start value for ivar
i2 integer maximum (or minimum) value for ivar
i3 integer increment of ivar

GOTO line;
 continue interpretation of input at the specified line
 number of the input file.

The commands described above can be used to prepare input data for any application. The following commands, however, are application dependent.

3.9.5 Commands Used for Geometry Description

GEOCMP nbody nzone nczone lenfpd lenma;
 nbody integer number of bodies used
 nzone integer number of zones used
 nczone integer number of c-zones used
 lenfpd integer length of floating point data needed
 lenma integer length of integer data needed

GEOCMP must be the first command used for geometry description.

Body definitions:

ARB k vv1...vv8 is1...is6;
 k integer body reference number
 vv1...vv8 8 vectors corners of the body
 is1...is6 6 integers surfaces of the body;
 each surface is described
 by a 4 digit integer,
 (digits representing the
 index of the corner vvi)

definition of an arbitrary hexahedron, defining the 8 corners and describing all 6 surfaces by 4 the corners each.

SPH k vv r;
 k integer body reference number
 vv vector midpoint of the sphere
 r real radius

definition of a sphere

RCC k vv hh r;
 k integer body reference number
 vv vector midpoint of the bottom surface
 hh vector height vector
 r real radius

defines a rectangular, circular cylinder starting at point vv; axis direction and height are given by hh; the radius is r.

REC k vv hh rr1 rr2;
 k integer body reference number
 vv vector midpoint of the bottom surface
 hh vector height vector
 rr1 vector mayor half axis of the ellipsis
 rr2 vector minor half axis of the ellipsis

defines a rectangular, elliptic cylinder starting at point vv. The cylinder axis direction and height are given by vector hh. The major and minor half axes of the ellipsis are given by the vectors rr1 and rr2.

TRC k vv hh r1 r2:

k	integer	body reference number
vv	vector	midpoint of the bottom surface
hh	vector	height vector
r1	real	radius at vv (bottom surface)
r2	real	radius at vv+hh (top surface)

defines a truncated, rectangular cone starting at point vv with radius r1. Axis direction and length are defined by vector hh. r2 gives the radius at the end of the axis (hh).

ELL kv vv1 vv2 b;

k	integer	body reference number
vv1	vector	location of focus f1
vv2	vector	location of focus f2
b	real	length of the major axis

defines an ellipsoid.

BOX k vv hh1 hh2 hh3;

k	integer	body reference number
vv	vector	location of one corner
hh1	vector	direction and length of the 1st edge
hh2	vector	direction and length of the 2nd edge
hh3	vector	direction and length of the 3rd edge

defines a box with 3 pairs of parallel surfaces. Starting from one corner, 3 vectors are used to define the direction and length of the edges of the box. hh1, hh2 and hh3 must be orthogonal.

WED k vv hh1 hh2 hh3;

k	integer	body reference direction
vv	vector	location of one corner (from which heights are defined)
hh1	vector	direction and length of 1st basis edge
hh2	vector	direction and length of 2nd basis edge
hh3	vector	height vector of the prisma

defines a wedge. A wedge is a rectangular prisma with a rectangular triangle used as basis surface. vv denotes the location of the rectangular corner of the wedge, hh1 and hh2 describe the orientation and size of the basic triangle and hh3 the height of the prisma. hh1, hh2 and hh3 must be orthogonal.

```

RPP      kxmin xmax ymin ymax zmin zmax;
          k      integer      body reference number
          xmin   real         minimum x coordinate
          xmax   real         maximum x coordinate
          ymin   real         minimum y coordinate
          ymax   real         maximum y coordinate
          zmin   real         minimum z coordinate
          zmax   real         maximum z coordinate

```

defines a rectangular (coordinate-axis) parallel epiped (brick).

The following commands define semi-infinite bodies.

```

P      k uu d;
        k      integer      body reference number
        uu     vector       normal direction of the plane
        d      real         distance from coordinate origin

```

defines a plane in space using Hesse form. All points lying beyond this plane (looking into normal direction) are inside the body.

Special planes are:

```

PX      k d;
PY      k d;
PZ      k d;
          k      integer      body reference number
          d      real         distance from coordinate origin
          uu     -            is defaulted to the x, y or z direc-
                               tion, respectively

```

To simplify the preparation of input we also defined a shifted Hesse form.

```

PS      k vv uu d;
          k      integer      body reference number
          vv     vector       shift vector
          uu     vector       normal direction of the plane
          d      real         distance from vv

```


Circular cylinders of infinit length have been implemented as:

```

C      k vv u r;
CX     k vv r;
CY     k vv r;
CZ     k vv r;
      k      integer      body reference number
      vv     vector       any point on the cylinder axis
      uu     vector       the direction of the cylinder axis
                          (may be defaulted by the command
                          verb to x, y or z direction)
      r      real         radius of the cylinder

```

END;

The body input must be terminated by the END command.

Commands used for geometry zone input:

```

ZONE   imed ireg code-zone [OR code-zone] [...] [GO subgo];
      imed integer        medium number
                          -1 for black holes
                          <-1 to suspend assignment to a
                          subsequent sub-geometry
      ireg integer        region number
                          if <0, 3 digit part of a 9 digit
                          compound region number
      code-zone one or more integer
                          body reference numbers
                          if >0, all points of the code-zone
                          lie within the body
                          if <0, all points of the code-zone
                          lie outside the body
      OR keyword          logical operator to join code-zones
                          together to one input zone
      GO keyword          indicates that the zone will be sub-
                          divided into subzones
      subgeo integer      identify the zone-card set to be used
                          for subdividing the zone

```

SUBGEO;

indicate the start of a new sub-geometry (zone-card set).
Zone card sets are sequentially numbered from 1 to 10.

ENDGEO;

indicates the end of the geometry description

GEOIN in, io:
 in integer unit from which a precompiled geometry
 input is read
 io integer unit for printed output

Commands used to test the geometry description: these commands can be used when the geometry is loaded using GEOIN.

LOOKZ vv;
 vv vector location of a point to be examined

LOOKZ will printout the point, the region and medium number and the internally used zone number.

RAY ss uu d;
 ss vector starting location of the ray
 uu vector direction of the ray
 d real length of the ray

RAY performs ray-tracing, starting at location vv in direction uu to the distance d.

Position, region number, medium number and zone number is printed out at vv, at each intersection point with a geometry boundary and at the end of the ray $vv+d*uu$.

The following geometry-test commands can be used on an interactive operating system providing the graphic software.

PICZ ss uu vv d;
 ss vector midpoint of the picture
 uu vector x-direction of the picture
 vv vector y-direction of the picture
 d real size of the picture

PICZ generates a picture plot of a plane cut through the geometry. The lower left corner of the plot refers to the space coordinates $ss-d*uu-d*vv$, the upper right corner to $ss+d*uu+d*vv$. The zone boundaries will be plotted.

PICM ss uu vv d;

PICR ss uu vv d;

PICM plots medium boundaries, PICR the region boundaries. PICZ, PICM, PICR will prepare a picture which can be completed interactively using the following commands.

POINT n;
 n integer number of points to be examined

The user is prompted for n points (input with the graphic cursor). At each point the zone region or medium number will be indicated.

GRNXTF:

the picture is completed and the next picture can be prepared. GRNXTF offers the picture for output on an additional device.

3.9.6 Commands Used in SIM

See also Section 3.7.1

DEFDET name, type, part, ud, ebin, abin, ir1, ir2;
 name variable 4 character name used to identify
 name all arrays associated with the
 detector
 type integer detector type
 1 - cascade yield
 2 - evaporation yield
 3 - total yield
 4 - energy deposition
 5 - scintillation light
 6 - forward surface current
 7 - forward surface flux
 8 - tracklength volume flux
 9 - energy passing surfaces
 prt real particle type
 0 proton 6 μ^-
 1 neutron 7 deuterons
 2 π^+ 8 tritons
 3 π^0 9 ^3He ions
 4 π^- 10 α
 5 μ^+
 ud vector detector reference direction
 ebin real energy bin limits descending
 array [MeV]
 abin real angle bin limits ascending
 array [$^\circ$]
 ir1 integer list of region numbers
 array (from region)
 ir2 integer list of region numbers
 array (to region)

3.9.7 Commands Used in STATIST

The STATIST commands are described in Section 3.8.2. Here we give only a summary of the command syntax.

```

STATIN  nv, nevt;
        nv      integer      number of variables
        nevt    integer      number of events

LOAD    unit, recordid, array;
        unit    integer      unit number of submission file
        record-id name      record name
        array name          variable name

SET      dest=srs1 [*srs2] [+...];
        dest    name          destination
                subscripted name
        srsi    name          operands
                subscripted name
                constant

DOT      dest=srs1*srs2 [(n1,n2,n3)];
        dest    name          destination matrix
        srsi    name          source matrices
        ni      integer      temporary dimensions

IF       srs1 symbol srs2;
        srs1    name          items to be compared
                subscripted name
                constant
        symbol  <            comparison operator
                =
                >

AND      srs1 symbol srs2;
        see above

OR       srs1 symbol srs2;
        see above

DROP;

LIST;

```

```

VETO      srs1 symbol constant;
          srs1 name
          subscripted name
          symbol <
              =
              >

```

Analysis Commands:

```

STATIST    ioplot, splot, iosas;
          ioplot PRINT      plot option
                  PLOT
          splot  real        plot size in cm
          iosas  integer      unit number for SAS output file

BIN         iv, from, to, nbin, label;
          iv      integer      variable index
          from    real         lower binning limit
          to      real         upper binning limit
          nbin    integer      number of binning intervals
          label   string       descriptive text in inverse commas

FIT         if, from, to;
          iv      integer      variable index
          from    real         lower limit for Gaussian fit
          to      real         upper limit for Gaussian fit

SCAT        ivx, ivy;
          ivx     integer      index of x-variable
          ivy     integer      index of y-variable

```

3.9.8 Commands Used for Graphic Output

CMD includes an interface to the KFA graphic software called GR software. Since this software cannot be installed at any other installation, a functional description of the GR sub-routines is given in Appendix D. We, however, provide some routines to simulate a subset of the GR software on a simple line printer.

```

GRSTRT;
          turn on graphic software

GREND;
          turn off graphic software

```

CHHEAD 'chart-title';
define the chart title

CHXAXS scale,from,to,'title';

CHYAXS scale,from,to,'title';
define the chart axis
scale may be LIN or LOG for linear or logarithmic
scaling of the axis
from lower limit for the axis
to upper limit for the axis
if 'from' or 'to' is not given, it will be
set automatically
"title" text to be written at the axis

CHCURV x,y,colour,symbol,line,'text';
define a
x real array of abscissa
y real array of ordinates
given as variable name or array constant
colour integer value defining the colour of the
curve
1 black
2 red
3 blue
4 green
symbol integer value defining the symbol to be
drawn at each point x_j, y_j
0 no symbol
1 symbol number i (maximum 13)
line 0 no lines between points
1 straight lines between points
'text' is a descriptive text which will be plotted
in the legend field of the chart.
The text appears in the same colour as the
curve and will be preceded by a piece of
line and the matching symbol.

CHBARS x,y,colour,symbol,line,'text';
define a bar-curve (histogram)
x real array of group limits
y real array of ordinates per group
colour integer value defining the colour
1 black
2 red
3 blue
4 green

symbol integer value defining a symbol to be placed
in the group midpoints
0 no symbol
i symbol number i (maximum is 13)

line 0 no bars drawn
1 draw bars

'text' function description to be placed in the
legend field of the chart

CHLGND colour,symbol,line,'text';
define a text to be placed in the legend field of the
chart

colour integer value defining the text colour
1 black
2 red
3 blue
4 green

symbol integer value defining a symbol to prefixed
to the text
0 no symbol
i symbol number i (maximum 13)

line integer value defining a piece of line as
prefix to the text
0 no line
1 draw line

'text' text to be placed in the legend field of the
chart

CHNEXT;

show current chart and clear chart-memory

CHART;

invoke a chart input panel.

The panel summarizes the above chart commands. It can
be used on VM/CMS systems and uses the VM/CMS editor
and system product interpreter REXX.

3.10 References Chapter 3

/1/ R. Rühle

RSYST, Modulares Programmsystem zur Reaktorberechnung
IKE-4-12 (1973)

/2/ C.E. Burgart

Capabilities of the MORSE Multigroup Monte Carlo Code in
Solving Reactor Eigenvalue Problems
ORNL-TM-3662 (1971)

/3/ W.R. Nelson, H. Hirayama, D.W.O. Rogers
The EGS4 Code System
SLAC-265, December 1985

/4/ R.L. Ford, W.R. Nelson
The EGS-Code Systems: Computer Programs for the Monte Carlo
Simulation of Electromagnetic Cascade Showers
SLAC-210, UC-32, June 1978

APPENDIX A

MODIFICATIONS OF THE ORIGINAL MONTE CARLO CODES USED IN HERMES

Below modifications to the original program versions contained in the HERMES system are described. The main emphasis was laid on better physics in the programs. Because these modifications are not published elsewhere, we give a detailed description here.

A.1 Modifications of HETC

A.1.1 Model for Elastic Scattering of Protons and Neutrons up to 20 GeV

(1) Introduction and Summary

Elastic scattering is usually not taken into account in HETC code calculations. While present versions of the code contain programming to allow the user to input elastic scattering cross sections (presently restricted to energies below 100 MeV and only for neutrons), such input data are not readily available. (Furthermore, the angular scattering distribution is assumed to be of the form $a+b\cdot\cos\theta$, which is very approximate.) Therefore, the elastic scattering capabilities of all present versions of the HETC code are incomplete, and almost all calculations are made with elastic scattering neglected, the assumption being that at high energies the angular change and energy loss from elastic collisions is small.

However, there are several situations related to accelerator shielding, visible energy in high energy physics calorimeters, heat deposition in materials etc. where elastic scattering effects may be important. Therefore, we have modified the HETC code to include in a general way the effects of high-energy elastic collisions to improve the physical models.

The approach used for arriving at neutron and proton high-energy elastic scattering cross sections and angular distributions for incorporation into HETC is described. Elastic cross-sections from several sources have been compared, and we have adopted neutrons values from the HIL0 library /1,2/ in the energy range from 15 MeV to 150 MeV, and values from the NASA library /3/ from 150 MeV to 20 GeV. For protons, cross sections from the NASA library are used above 100 MeV, and elastic scattering for protons at lower energies is neglected. An approximate expression based on the optical model is used for the scattered angular distributions. We have made optical model

calculations and comparisons with experimental data to check the validity of this approximate angular distribution, and find it to be sufficiently accurate. Also, the details of the HETC code modifications to implement elastic scattering are given.

(2) Assessment of Elastic Cross-Sections

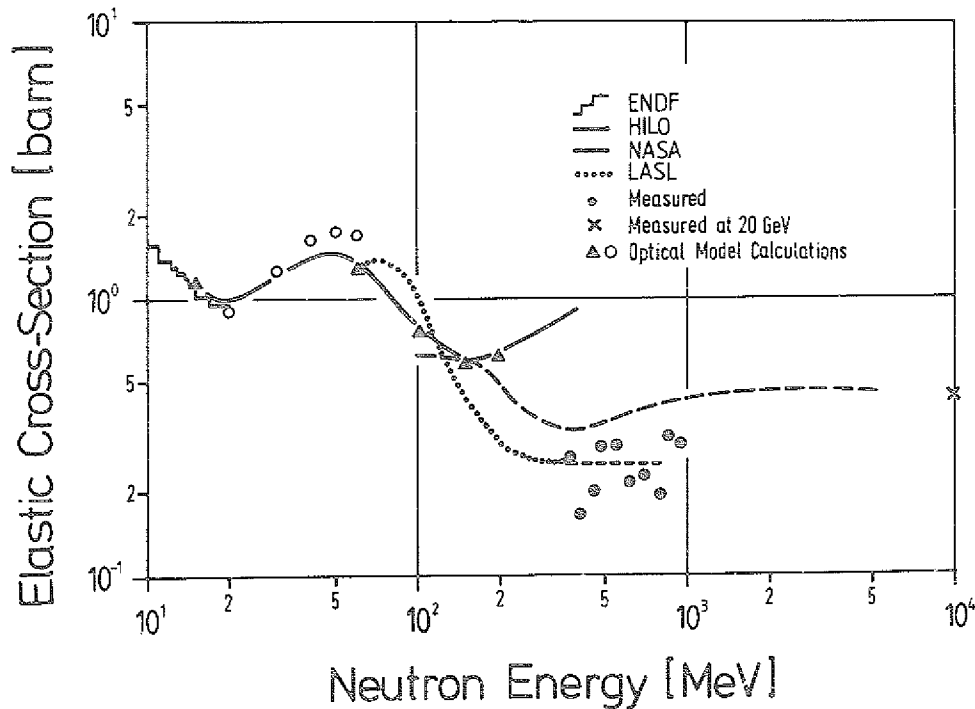


Fig. A.1: Comparison of neutron elastic cross sections for iron from various sources

Elastic cross-sections for neutrons have been collected from several sources for comparison. For example, Figure A.1 compares several sets of data for iron including: (a) Cross-sections from the low-energy (≤ 19.6 MeV) DNA library /4/, which is based on ENDF data. (b) Cross sections from the ORNL high-energy (≤ 400 MeV) HILO library /2/. These cross-sections were generated using the ORNL optical model code GENOA. (Explicit values for σ_{el} are not available from the library; these were provided to us by Dr. Alsmiller of ORNL.) (c) Cross sections from a NASA compilation for the energy range 100 MeV to 22.5 GeV /3/. These are theoretical estimates from numerical solutions of the coupled-channel Schroedinger equations. (d) Cross-sections from the LASL multigroup library /5/. (This library does not take into account elastic scattering; the elastic cross-section shown is the difference between the library values for the total and non-elastic cross sections.)

(e) Results from optical model calculations reported by Wilson /5/. (f) The measurements of Schlimmerling et al. /6/ in the ~400-1000 MeV range, and the measurements of Bellettini, et al., /7/ at 20 GeV. (g) Optical model calculations which we have made using the A-THREE code /8/ and the parameters used are also described later.

The main conclusion from Figure A.1 is that there are large differences in the data in the $\sim 10^2$ - 10^3 MeV range. In particular, the HILO values appear incorrect ≥ 150 MeV. (This is also the case for other elements. (This is meanwhile corrected by Alsmiller et al. /9/ in the HILO86 release.) To check this, we have made optical model calculations using the A-THREE code for the same optical model parameters as used by ORNL in generating the HILO library, and we obtained essentially the same results as ORNL. Thus, this discrepancy in the HILO high-energy elastic cross sections is not attributable to the optical model code used by ORNL. The reason for the apparent error in the HILO cross sections at ≥ 150 MeV is probably that the optical model parameters used by ORNL, which were taken from the work of Becchetti and Greenless /10/ and based on fits to data < 50 MeV are not applicable at high-energies.

The A dependence of the HILO elastic scattering cross section at high energies also differs substantially from the NASA values. For example, at 400 MeV, $\sigma_{el} \propto A^{2/3}$ for the HILO library, whereas $\sigma_{el} \propto A^{0.94}$ for the NASA library. (The NASA values are in good agreement with the measured /7/ dependence of $\sigma_{el} \propto A^{1.04}$ for energies ≥ 10 GeV.)

Note from Figure A.1 that the HILO and NASA cross sections agree at 150 MeV. At very high energies, the NASA cross sections are in very good agreement with the measurements of Bellettini, et al. /7/. Therefore, for incorporation into HETC to treat neutron elastic-scattering, we have made a tabulation of σ_{el} vs. E values for various A using HILO values in the energy range from 15 to 150 MeV and NASA values from 150 MeV to 20 GeV (with linear E interpolation and log-log A interpolation). The values used for both neutrons and protons are given in Tables A.1-A.3.

Table A.1: Input data used for high-energy neutron elastic cross sections (millibarns)

Energy (MeV)	Element									
	He	C	O	Al	A	Fe	Br	Ag	Ba	Pb
100	67	182	225	358	499	623	786	987	1162	1535
125	61	171	217	345	483	614	782	984	1161	1537
150	58	162	210	335	469	605	775	978	1155	1535
175	49	141	187	299	423	558	729	928	1106	1487
200	38	113	154	248	355	482	646	834	1006	1383
225	33	100	137	222	320	441	599	778	945	1316
250	30	90	125	202	292	406	558	729	892	1255
300	24	75	107	173	253	356	497	654	807	1152
400	23	70	98	161	235	331	464	611	757	1088
600	29	86	117	192	280	382	525	684	840	1184
1000	36	102	136	220	319	425	574	742	903	1254
2000	41	114	151	242	347	461	615	790	955	1314
5000	40	110	147	237	339	453	607	782	946	1306
10000	37	102	138	222	319	430	581	752	913	1268
22500	34	95	129	206	298	406	552	716	874	1223

Table A.2: Input data used for low-energy neutron elastic scattering cross sections (barns)

Energy (MeV)	Element						
	¹⁰ B	C	N	O	Na	Mg	Al
15	0.94	0.92	0.90	0.98	0.88	0.90	1.09
20	0.95	0.95	0.97	1.04	1.00	1.14	1.18
25	0.94	0.94	1.00	1.09	1.10	1.20	1.24
30	0.88	0.88	0.99	1.06	1.16	1.22	1.28
40	0.75	0.75	0.92	0.94	1.14	1.16	1.23
50	0.63	0.61	0.78	0.80	1.03	1.06	1.16
60	0.50	0.50	0.63	0.68	0.84	0.88	0.93
80	0.31	0.37	0.40	0.44	0.56	0.60	0.63
100	0.22	0.25	0.28	0.31	0.40	0.43	0.46

Energy (MeV)	Element						
	Si	S	K	Ca	Cr	Fe	Ni
15	0.72	0.93	1.08	0.89	1.10	1.18	1.32
20	1.00	1.08	1.24	0.94	0.95	0.98	1.00
25	1.08	1.19	1.34	1.07	1.10	1.09	1.08
30	1.23	1.25	1.31	1.25	1.30	1.27	1.24
40	1.27	1.30	1.38	1.51	1.63	1.65	1.64
50	1.18	1.23	1.28	1.53	1.77	1.80	1.83
60	1.01	1.04	1.15	1.17	1.30	1.34	1.38
80	0.67	0.73	0.82	0.85	0.96	1.01	1.05
100	0.48	0.54	0.61	0.63	0.73	0.78	0.81

Table A.3: Input data used for proton elastic collisions
(millibarns)

Energy (MeV)	Element									
	He	C	O	Al	A	Fe	Br	Ag	Ba	Pb
100	67	182	225	365	524	639	811	1015	1204	1588
125	61	171	217	354	511	632	809	1013	1202	1583
150	58	162	210	342	495	622	802	1006	1196	1576
175	49	141	187	306	445	574	755	956	1054	1527
200	38	113	154	253	370	492	665	854	1036	1416
225	33	100	137	226	331	449	614	794	970	1344
250	30	90	125	204	301	412	576	742	910	1276
300	24	75	107	175	258	360	502	659	814	1158
400	23	70	98	161	236	331	464	610	575	1085
600	29	86	117	190	276	377	516	672	822	1158
1000	36	102	136	220	318	425	573	740	900	1251
2000	41	114	151	243	350	463	621	797	966	1330
5000	40	110	147	237	342	455	612	788	955	1319
10000	37	102	138	222	321	432	584	756	1009	1277
22500	34	95	129	207	298	406	553	717	876	1225

(3) Optical Model Calculations

To make predictions of σ_{el} and $d\sigma_{el}/d\Omega$, we have obtained an optical model code called "A-THREE" /8/ from Brookhaven National Laboratory and made calculations for various cases. The basic features of the code and the model parameters which have been used here are discussed briefly below.

(a) The A-THREE Optical Model Code

The A-THREE code predicts the total cross-section, elastic cross-section, and the angular distribution of the scattered particle for charged or uncharged incident particles. Basically, the code numerically solves the center-of-mass Schrödinger equation for a nuclear potential having the form

$$V(r) = V_{re}(r) + i V_{im}(r) + V_{so}(r) \vec{l} \cdot \vec{\sigma} + V_{coul}(r),$$

where the terms represent the real, imaginary, spin-orbit, and Coulomb parts of the potential. The following forms are allowed for specifying these potentials:

$$V_{re}(r) = -V_0 f_1(r) - V_1 f_2(r)$$

$$V_{im}(r) = -W_0 f_1(r) - W_1 g_1(r) - W_2 g_2(r)$$

$$V_{so}(r) = (h/m_\pi c)^2 V'_{so} h(r)$$

$$V_{coul}(r) = h_2(r)$$

where various functional forms for f , g , and h are allowed, such as Woods-Saxon, Gaussian, or arbitrary forms which are input point-by-point.

(b) Optical Model Parameters

The A-THREE code does not include values for the above parameters - these are supplied by the user for particular cases of interest, and are usually obtained from studies which have been made to infer optical model parameter fits from comparisons with experimental data.

We have made A-THREE code calculations using the parameters given by Becchetti and Greenless /10/. In terms of the above A-THREE variables, the Becchetti and Greenless parameters are (in MeV):

$$V_0 = 5.3 - 0.32E - 24(N-Z)/A; \quad V_1 = 0$$

$$W_0 = 0; \quad W_1 = 0.22 \cdot E - 1.6; \quad W_2 = 13 - 0.25E - 12(N-Z)/A$$

$$V_{so} = 6.2$$

According to Becchetti and Greenless, if the energy and target variables are such that any of the above parameters are negative, they should be set to zero. For $f_1(r)$ and $g_1(r)$ a Woods-Saxon potential is used, and for $g_2(r)$ and $h(r)$ the derivation of a Woods-Saxon form is used - e.g.:

$$f(r, R, a) = [1 + \exp(r-R)/a]^{-1}$$

$$g(r, R, a) = 4a(df/dr)$$

For incident charged particles, a uniform charge distribution out to radius R_C is assumed for the Coulomb potential:

$$V_{coul}(r) = (Z \cdot z \cdot e^2 / 2R_C) \cdot [3 - (r^2/R_C^2)], \text{ for } r \leq R_C$$

$$= Z \cdot z \cdot e^2 / r, \text{ for } r \geq R_C$$

(Z, z are the charges of the incident particle and target).

W_2 , V_{SO} ; and four sets of R and a in the expressions for f_1 , g_1 , g_2 , and h .

It is important to note that the parameters given by Becchetti and Greenless are based on fits to experimental data for target nuclei having $A > 40$ and neutrons and protons with $E < 50$ MeV. However, they were used up to 400 MeV by ORNL in generating the HILO neutron cross section library. Note from the above expressions that V_0 , W_1 , and W_2 all become zero for $E \geq 170$ MeV (using for example N , Z , and A for iron), and so both the real and imaginary terms become zero. Thus, the reason for the discrepancy in the old HILO elastic cross sections ≥ 150 MeV indicated in Figure A.1 appears to be because the optical model parameters used are not valid at such energies.

(4) Assessment of Angular Distributions for Elastic Scattering

Several approaches have been considered for obtaining the angular distributions for the elastically scattered neutrons or protons, namely: an approximate optical model distribution and a semi-empirical formula based on measurements at very high energies. While in principle the A-THREE code could be used to generate such angular distributions for use in HETC, there are two practical difficulties: (1) optical model parameters are not available for A-THREE input over the wide energy and target mass range of interest, and (2) a very large volume of data would need to be generated and processed. Therefore, the approach we have used is to test the applicability of the simple formulae by making comparisons for a small number of selected cases with A-THREE optical model calculations and, where available, experimental data. It should be noted that these simple formulae are needed only for predicting the shape of the angular distribution. That is, the absolute magnitude of the differential scattering cross sections is not needed for the Monte Carlo selections, as the normalization is provided by the σ_{el} values discussed in the previous section.

(a) Approximate Optical Model Distribution

For the simplified special case of treating the nucleus as a black disk of radius R diffracting the deBroglie wave length of the incident particle, a rather simple form for the elastic scattering differential cross section is predicted by the optical model (e.g., /11/):

$$d\sigma/d\Omega \sim R^4/\lambda^2 \left(\frac{J_1 \cdot [2 \cdot R \cdot \sin(\theta/2)/\lambda]}{2 \cdot R \cdot \sin(\theta/2)/\lambda} \right)^2$$

where

- θ = center-of-mass scattering angle
 J_1 = Bessel function of first kind, first order
 R = $r_0 \cdot A^{1/3} + \lambda$
 λ = $h \cdot c / p \cdot c$
 hc = 1.9732×10^{-11} MeV cm
 pc = $[E \cdot (E + 2m)]^{1/2}$

The value for r_0 in the above formula is somewhat of an adjustable parameter. We have used $r_0 = 1.4$ Fermis for all of the results shown here.

Figures A.2 and A.3 indicate how the angular distributions predicted by the above approximate formula vary with target nucleus and neutron energy.

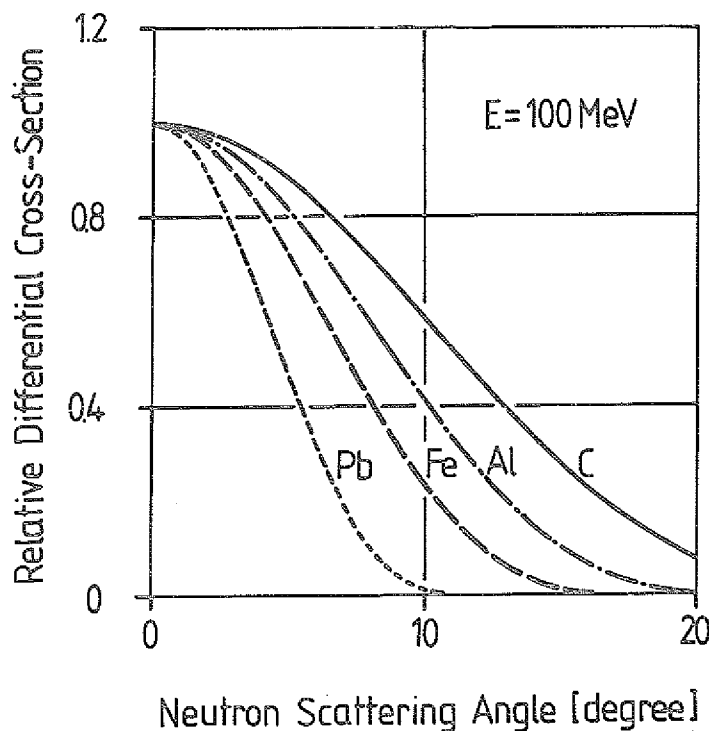


Fig. A.2: Predicted angular distributions of elastically scattered 100 MeV neutrons with various target nuclei

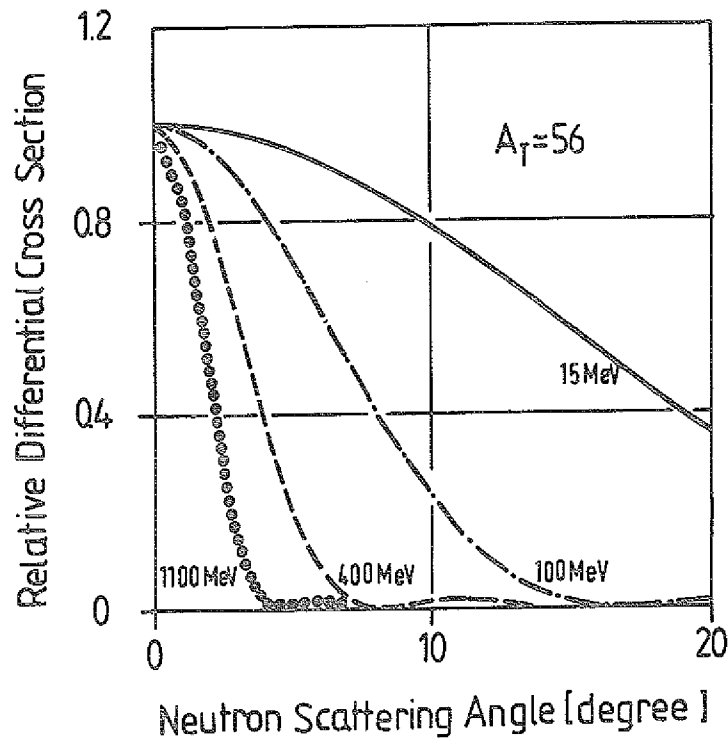


Fig. A.3: Predicted relative angular distributions of elastically scattered neutrons of various energies from iron nucleus

We have made some optical model calculations of angular distributions using the A-THREE code and the Becchetti and Greenless parameters to compare with the approximate formula. Comparisons for 15 and 100 MeV neutrons scattering on iron are shown in Figure A.4.

We have also compared the approximate angular distribution formula with the elastic scattering distributions measured by Bratenahl et al., /12/ for 84 MeV neutrons incident on Al, Cu, and Pb, and good agreement is obtained (Figures A.5-A-7).

Therefore, this approximate optical model expression for angular scattering distributions appears adequate for the energy range where the tests have been made (≤ 100 MeV).

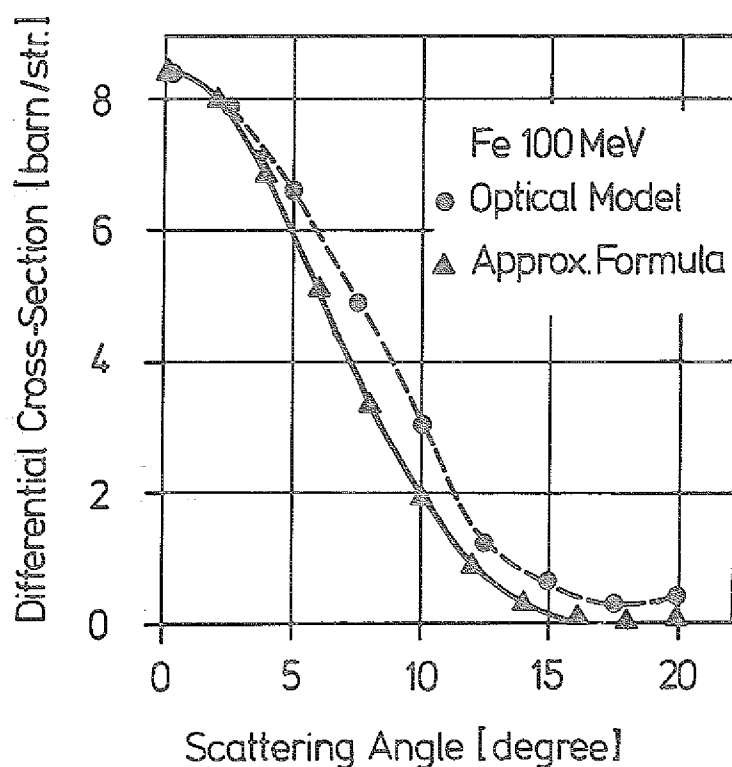
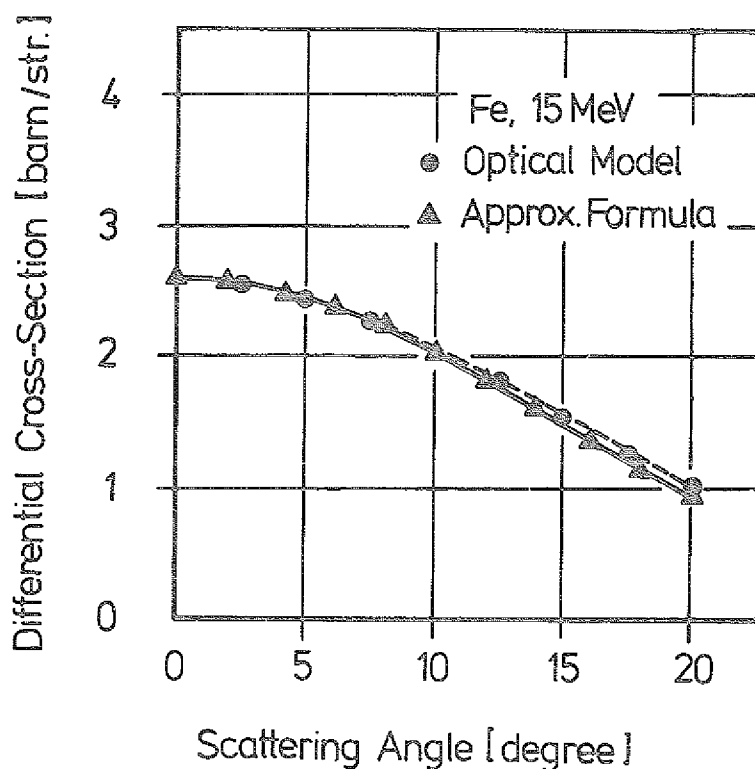


Fig. A.4: Comparison of angular distributions for 100 and 15 MeV neutrons elastically scattering from iron. Optical model results were computed using the A-THREE code and the potentials of Becchetti and Greenless. Distributions predicted by approximate formula normalization to optical model calculations at 0°

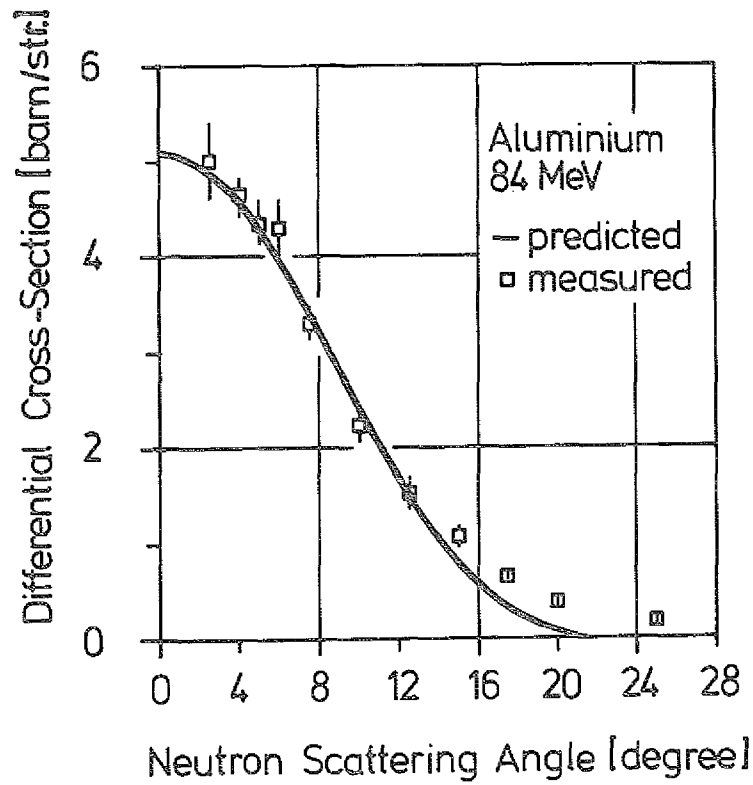


Fig. A.5: Predicted angular distribution of elastic scattering of 84 MeV neutrons compared with the measured data of Bratenahl, et al.

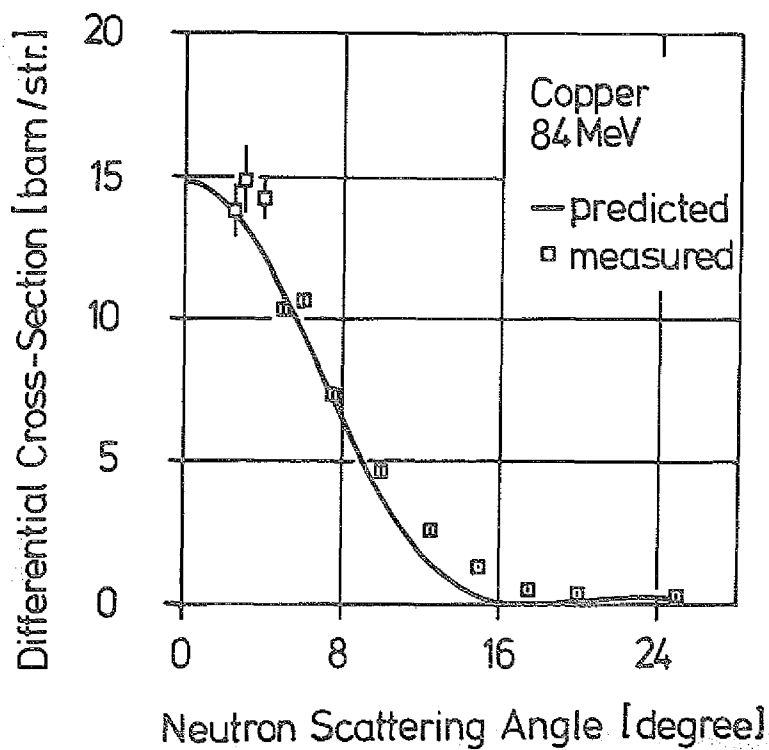


Fig. A.6: Like Figure A.5 but for copper

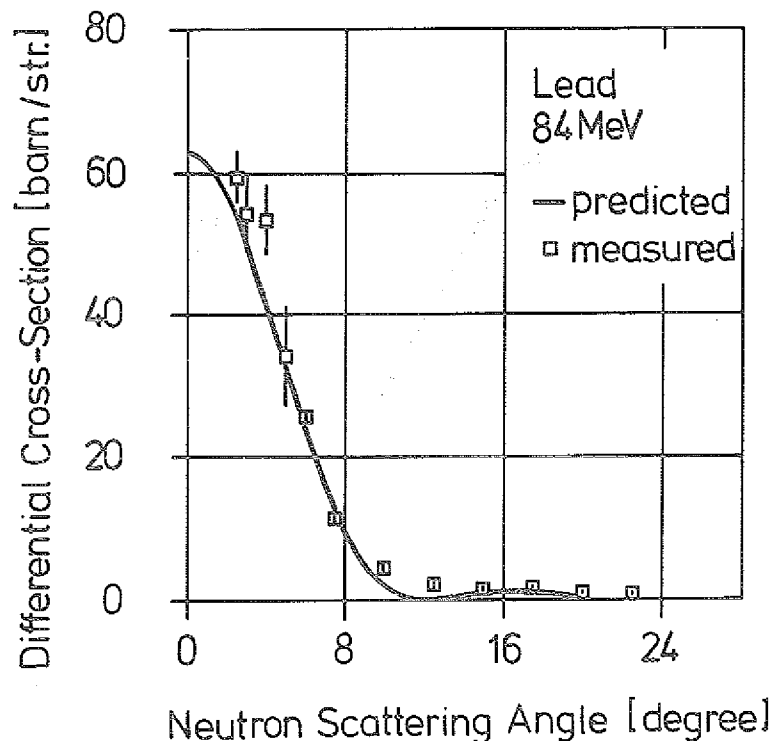


Fig. A.7: Like Figure A.6 but for lead

(b) Empirical Elastic Scattering Angular Distribution

The question arises as to whether the above angular distribution is accurate at higher (≥ 100 MeV) energies. Experimental data on angular distributions in the ~ 100 MeV to few GeV energy range are almost nonexistent. However, one series of measurements has been made by Bellettini et al. /7/ for 20 GeV protons on various target nuclei. Ranft /13/ has arrived at a semi-empirical formula to describe these data, which has the following form:

For $A < 62$,

$$d\sigma/d\Omega \sim A^{1.63} \exp(14.5 \cdot A^{0.66} \cdot t) + 1.4 \cdot A^{0.33} \cdot \exp(10t),$$

and for $A > 62$,

$$d\sigma/d\Omega \sim A^{1.63} \cdot \exp(60 \cdot A^{0.33} t) + 0.4 \cdot A^{0.40} \cdot \exp(10t),$$

where

- t = momentum transfer in $(\text{GeV}/c)^2 = -2p^2(1-\cos\theta)$
- p = momentum of scattered particle
- θ = center-of-mass scattering angle.

(The total elastic cross section based on these measurements at 20 GeV is $\sigma_{el} \sim 6.38 \cdot A^{1.04}$ mb, which is in good agreement with the values of the NASA data.)

(c) Comparison of Angular Distribution Formulae

Example comparisons of the approximate optical model angular distribution with the empirical scattering formula are given in Figs. A.8-A.11 for iron, for neutron energies of 100, 250, 400, and 1100 MeV.

Comparisons show that the distributions predicted by the two formula are very similar for all energies and target nuclei. The largest difference is at low energy (100 MeV). As shown in Section A.1.1.4a) the approximate optical model formula gives good agreement with experimental data at 100 MeV. Therefore, there is some basis for adopting this formula at low energies, and since there is little difference between the two formula at high energies, we have incorporated the approximate optical model distribution into HETC for predicting scattering distributions at all energies.

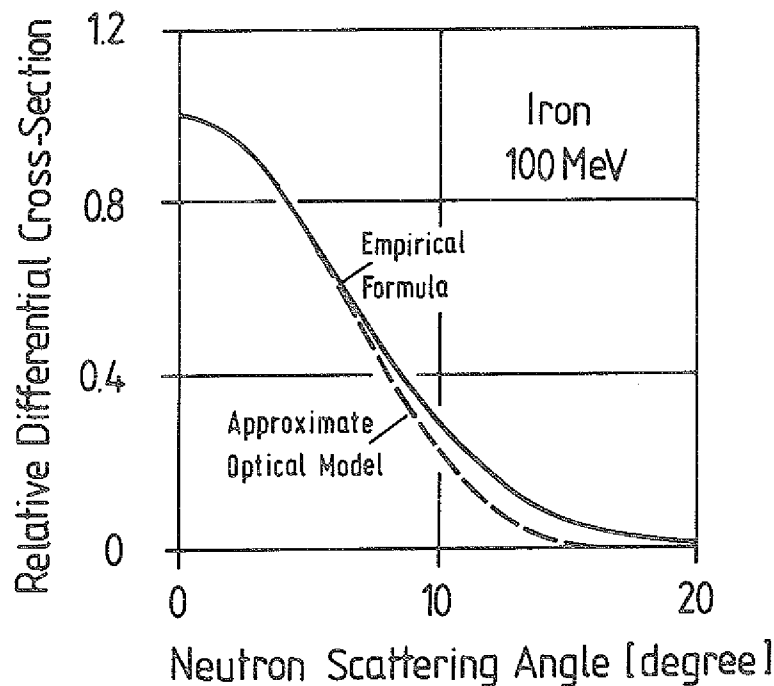


Fig. A.8: Elastic scattering angular distributions for iron, 100 MeV

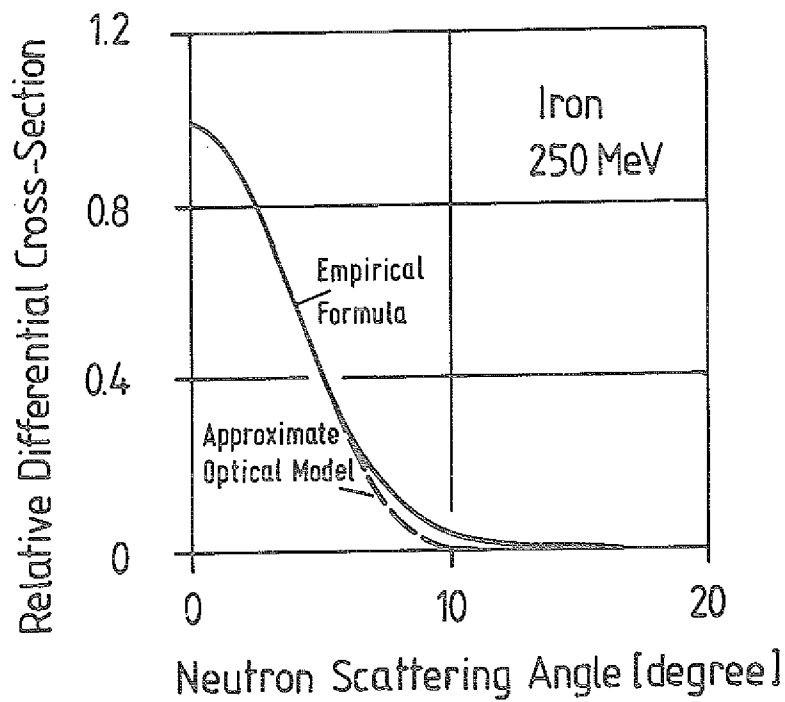


Fig. A.9: Elastic scattering angular distributions for iron, 250 MeV

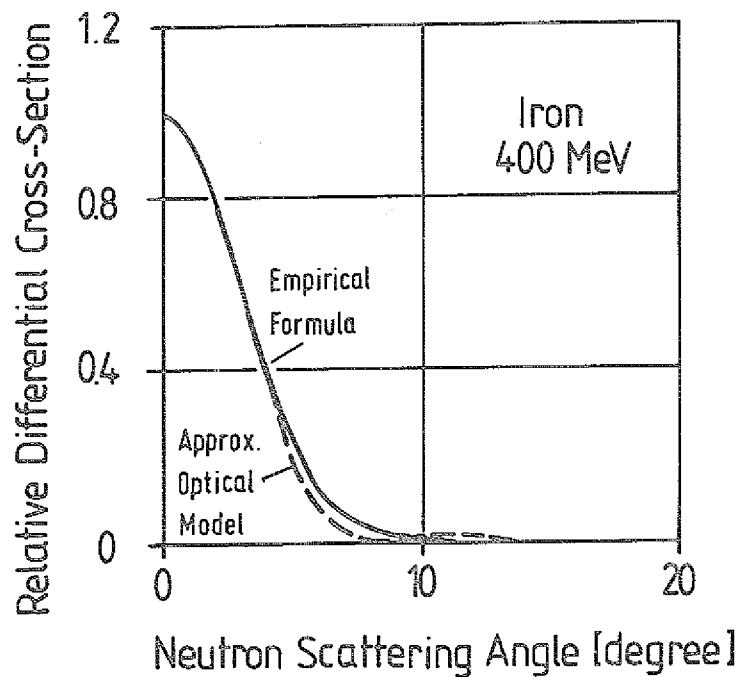


Fig. A.10: Elastic scattering angular distributions for iron, 400 MeV

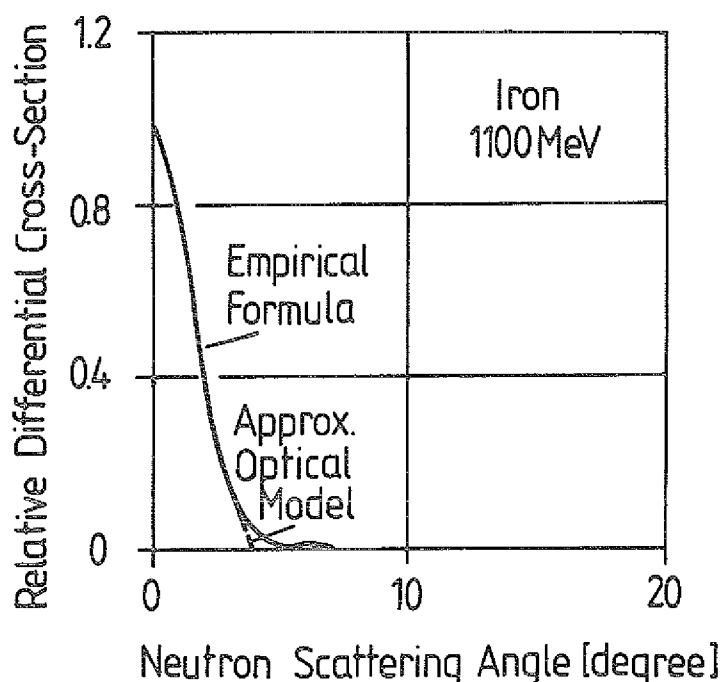


Fig. A.11: Elastic scattering angular distributions for iron, 1100 MeV

(5) Implementation of Elastic Scattering into the HETC Code

(a) Elastic Cross Sections Used

For computing elastic scattering in HETC, we have incorporated neutron cross-sections from the HILO library /2/ in the energy range from 15-100 MeV, and values from the NASA compilation /3/ at higher energies. The elastic cross section values used as HETC input are given in TABLES A.1-A.3, and the energy dependence is illustrated in Fig. A.12 for neutrons. Log-log interpolation of the data base values is used for both energy and target mass number interpolation. Fig. A.13 illustrates the target mass interpolation and extrapolation for several energies.

For protons, elastic cross-sections from the NASA compilation /3/ are used. These cross-sections are available only for ≥ 100 MeV, and elastic scattering by protons below 100 MeV is neglected in HETC. This is not an important approximation for most applications because protons in this energy range predominantly slow down and stop due to ionization energy losses rather than undergo nuclear collisions. The proton cross section values are essentially the same as for neutrons, but separate values for protons have been incorporated.

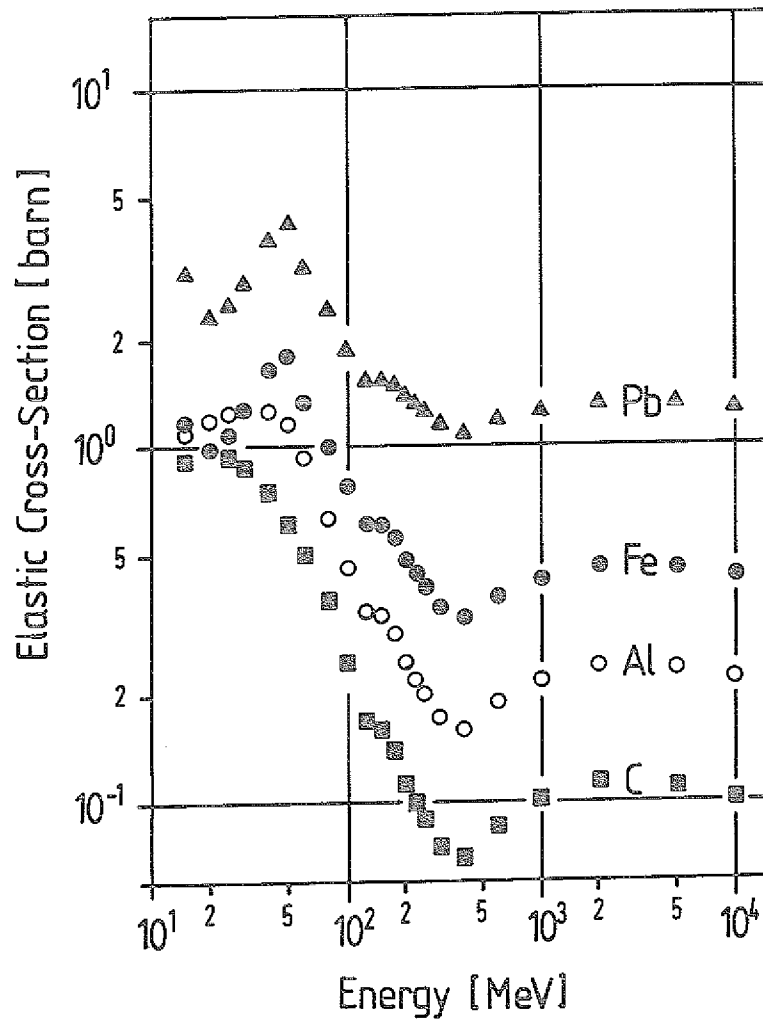


Fig. A.12: Energy dependence of neutron elastic scattering cross-sections (in barn) used as input, examples are shown for carbon, aluminum, iron, and lead nuclei.

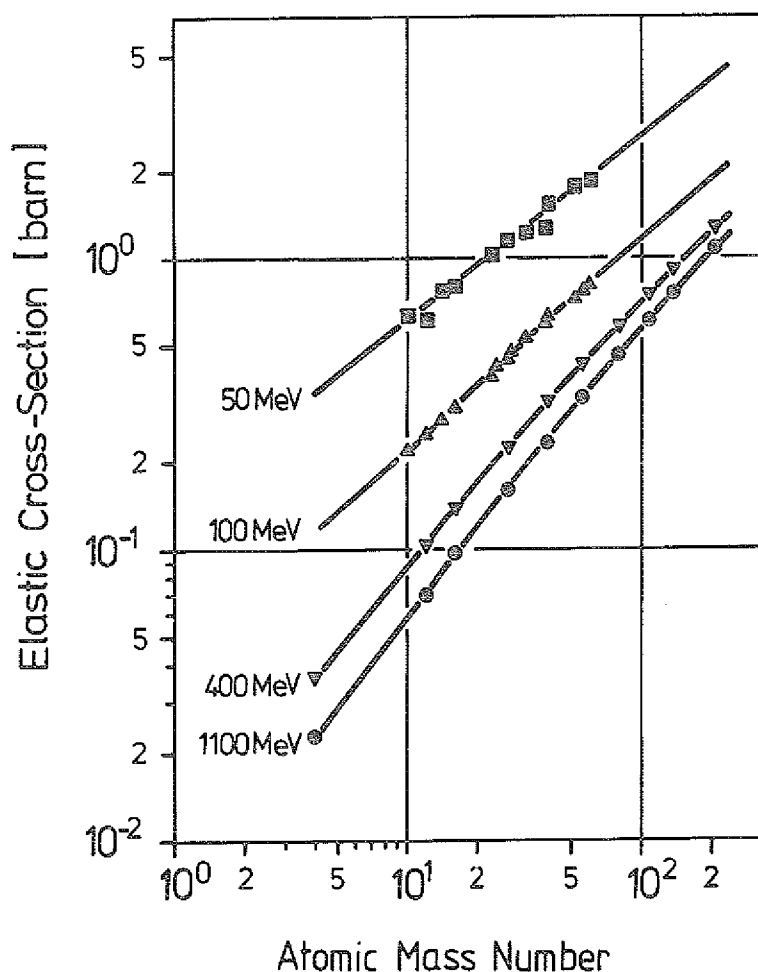


Fig. A.13: Target mass dependence of neutron elastic cross-section (barn) for several energies. The points are used in the data base, and the lines show the interpolation and extrapolation used

(b) Angular Distribution

The approximate optical model angular distribution formula discussed in Section A.1.1.4 is used for both neutron and proton scattering.

To determine the neutron or proton scattering angle, a random selection is first made of the variable x from the probability distribution function

$$f(x) \propto [J_1(x)/x]^2$$

where a series expansion is used for evaluating the Bessel function:

$$J_1(x)/x \sim 1/2 - x^2/16 + x^4/384 - x^6/18432$$

where the range of x is from 0 to the first zero of J_1 , which is $x_{\max} = (0.610)2\pi$. Random selections are made from a cumulative histogram of $f(x)$ from $x = 0$ to $x_{\max}$.

The angle (center-of-mass cosine) for a nucleon of energy E scattering from a nucleus of mass A is then (from Section A.1.1.4):

$$\cos\theta = 1 - x^2 \cdot 1/(2 \cdot [7.095 \times 10^{-3} \cdot A^{1/3} \cdot \sqrt{E(E+1878)} + 1]^2)$$

(c) HETC Code Modifications

The option for using elastic scattering is specified by setting a new input variable IELAS with the meaning:

```
IELAS = 0, don't use elastic scattering
        = 1, use elastic scattering for
              neutrons only
        = 2, use elastic scattering for
              neutrons and protons.
```

This is the only input variable relevant to elastic scattering; all cross-section values are incorporated in the code in data statements.

For neutrons, all elastic scattering collisions are "real". For protons, both "real" and "pseudo" collisions can occur for elastic collisions. Pseudo collisions are allowed for protons because the proton elastic-cross-section varies between events at different spatial points due to ionization. For proton elastic collisions, the same formalism as for non-elastic collisions is used. Thus, the path length is selected based on the maximum cross section with respect to energy. Then for a collision at the end of the path, the probability of a "real" collision is the ratio of the cross-section at the energy where the collision takes place to the maximum cross section. Using this method, the correct energy dependence of the cross section is preserved along the proton flight path.

If a real elastic collision occurs, the A and Z of the residual nucleus is set to be the same as the target nucleus, the number of "cascade" particles (NOPART) is set to 1, the evaporation particle array (NPART(I)) is zeroed, and EREC is set to the kinetic energy of the recoil nucleus. As a key to readily denote elastic scattering collisions, the excitation energy of the residual nucleus (which by definition is zero for elastic scattering) is set to -9999. Thus, for collision records, real

elastic scattering events can be identified by $NCOL=2$ and $EX=-9999$. (For pseudo collision records, no distinction is made between non-elastic and elastic collisions).

Deleted Subroutines

The following subroutines formerly used in treating elastic scattering have been removed: RELAS, CK, and SIGELS. Also, the common block ELSTIC, as well as all former input variables referring to elastic scattering, have been removed.

Modified Subroutines

INPUT: all reads of former elastic scattering removed, and calls two new subroutines ESNEUT and ESPROT (described below) added to get elastic-cross sections.

GETFLT: adds elastic cross section to non-elastic cross-section to get total cross-section for selecting flight path.

DKLOS: includes elastic cross-section to get proper total cross section for collision event selection.

CASCAD: determines whether collision is elastic; and if so, determines element struck, and calls new subroutine KINELS for kinematics calculation to get post-collision parameters.

ERUP: this subroutine usually increments a counter (NEGEX) if the residual excitation energy (EX) is negative; since the key for elastic scattering is $EX=-9999$, the NEGEX counter is not incremented for elastic collisions.

New Subroutines

SELCST: selects $\cos\theta$ for the elastically scattered nucleon by calling subroutine SELJ1 to select the parameter x ; function FX computes $[J_1(x)/x]^2$

ESNEUT: provides σ_{e1} arrays for neutrons; the values for σ_{e1} from the HILO library are stored in subroutine HILONX and σ_{e1} from the NASA library are stored in subroutine NASANX.

ESPROT: provides σ_{e1} arrays for protons; calls subroutine NASAPX to obtain σ_{e1} values based on the NASA library for protons.

KINELS: computes elastic scattering kinematics; given $\cos\theta$ (center-of-mass) for scattered nucleon, incident nucleon energy, and A of struck nucleus, this subroutine returns the scattered nucleon energy and direction cosines, and the recoil nucleus energy.

PSGELS: computes macroscopic elastic scattering cross-section for each element of medium; performs energy interpolation to get cross-section at end-of-path energy.

NSGELS: computes macroscopic cross section for neutrons, for each element in medium and total for medium.

A.1.2 Evaporation Model Modifications

The standard version of the HETC code (i.e., the version distributed by RSIC) contains version 4 of the EVAP evaporation model. The updated version described here will be referred to as EVAP-5

(1) Updated Input Data

In the evaporation model calculations, atomic masses are used in determining binding energies. These mass data (in terms of mass excesses) are stored on tape, read as input at the start of each transport calculation, and stored internally as the WAPS array, as defined in APPENDIX B.

EVAP-4 uses the evaluated 1964 Atomic Mass Table values /14/. At present the latest evaluated data are the 1977 Atomic Mass Evaluation by Wapstra and Bos /15/, and it is these data that have been put on the input data tape for EVAP-5.

In the standard EVAP-4, the WAPS (I,J) array was dimensioned WAPS (250,20), with I equal to the mass number of the nuclide J an index defining a particular isotope for a given A, and the following algorithm for retrieval:

$$J = A - 2Z - J' + 10$$

$$J' = (A-1)/(1+124/A^{2/3})$$

It was found that this method did not properly represent the new data. That is the Z and A range for the above method did not cover the range of the input values. In EVAP-5, the WAPS array is dimensioned WAPS (25,11) and WAPS(I,J") denote nuclides with I=A (as before) but J"=1 represents a "starting" Z value for this A. Then WAPS (I,1) corresponds to $Z_{\min}$, WAPS(I,2) corresponds to $Z_{\min}+1$, etc. These ten Z values for each A then allow all of the Mass Table values to be contained in the WAPS array, as illustrated in Figs. A.14-A.16.

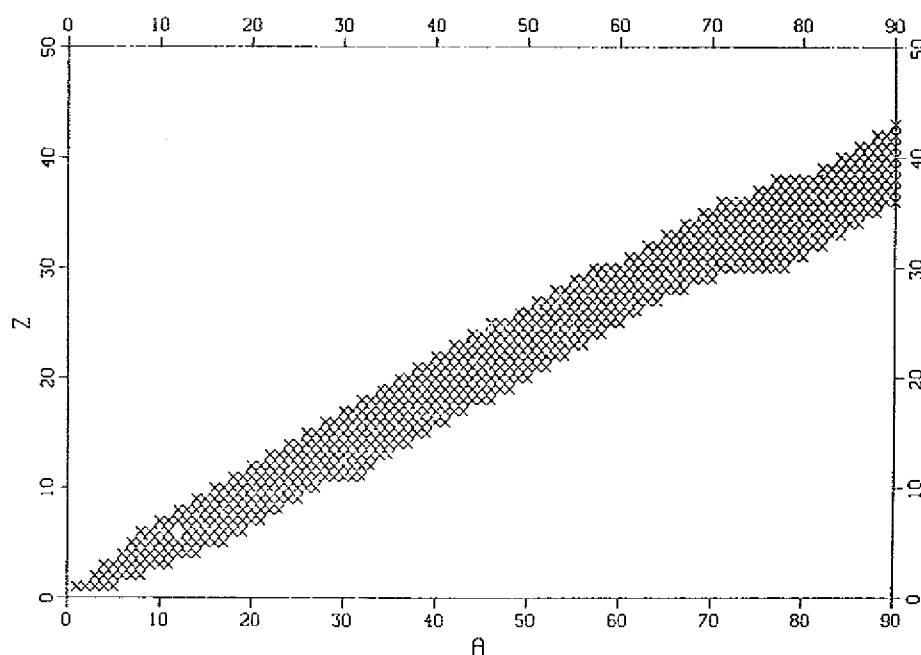


Fig. A.14: Nuclide A and Z range covered by updated input data for evaporation model. The symbols denote nuclides for which data are available and stored on the input tape. (Data for $A=1-90$ shown above; data for higher A in Figures A.15 and A.16)

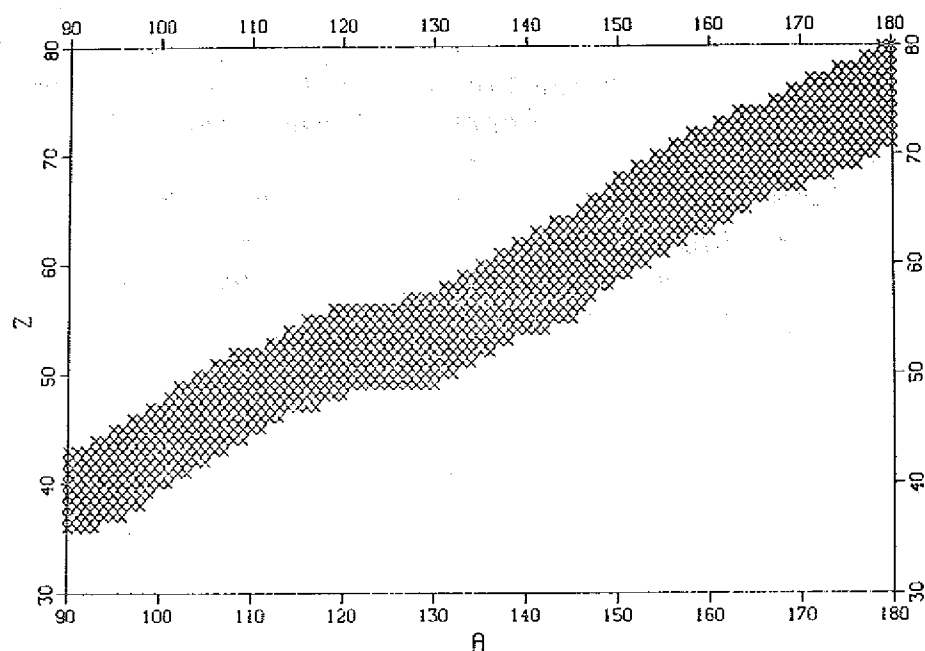


Fig. A.15: Nuclide A and Z range covered by updated input data for evaporation model. The symbols denote nuclides for which data are available and stored on the input tape. (Data for $A=90-180$ shown above, data for higher A in Figure A.16)

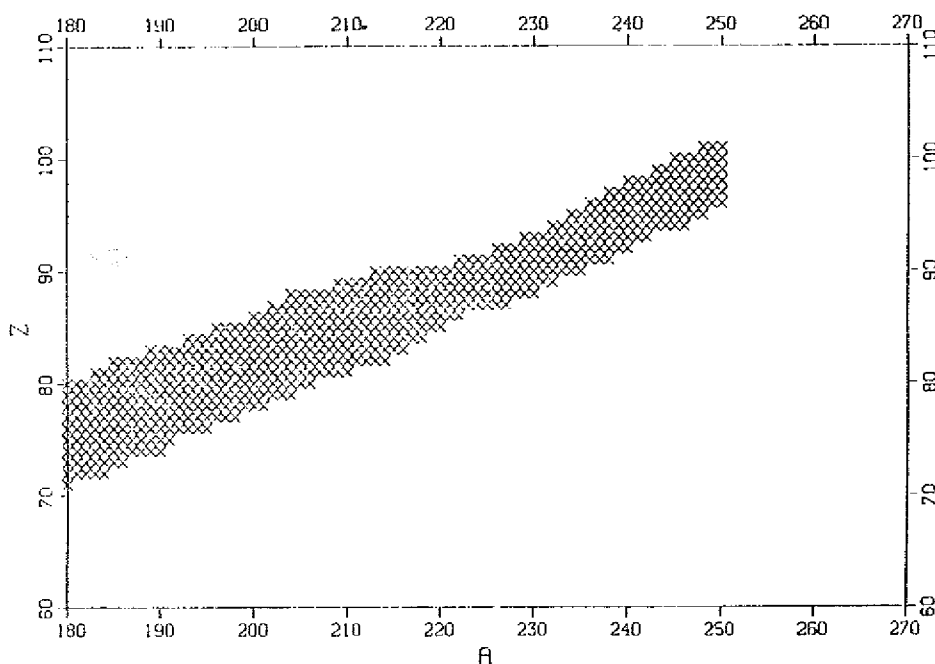


Fig. A.16: Nuclide A and Z range covered by updated input data for evaporation model. The symbols denote nuclides for which data are available and stored on the input tape (Data for A=180-250 shown above)

Another change was made in the procedure used when a nuclide mass is required that is outside of the range of the input data. The binding energy calculation involves the mass difference for two nuclides. In EVAP-4 if input data were not available for one or both nuclides, the Cameron semi-empirical mass formula (discussed in APPENDIX B) was used. Thus, in computing the mass difference, one mass could be computed from the mass formula and an input value used for the other nuclide. This can lead to what appear to be unrealistic mass differences. Thus, in EVAP-5, a consistent procedure is used - i.e., if one nuclide is outside of the range of the input data, both nuclide masses are computed using the mass formula. A new function routine,

```
FUNCTION QNRG (A1,Z1,A2,Z2)
```

computes the mass differences using this method.

(2) Level-Density Options Added

In EVAP-4 the level density parameter is computed as

$$a = A/B_0 (1 + Y\Delta^2/A^2)$$

with $B_0=8$, $Y=1.5$, and $\Delta=A-2Z$. The quantity in brackets differs little from unity, so the method assumes a linear mass dependence,

$$a \sim A/B_0 \text{ MeV}^{-1}.$$

From experiments on nuclear level spacing, it is observed that the level density is strongly influenced by nuclear shell structure, with the densities for magic or nearly magic nuclei several orders of magnitude lower than for mid-shell nuclei /16/. Therefore, an input option has been added to EVAP-5 to use a method for specifying the level density parameter which takes into account shell effects.

Baba /16/ has compiled level densities from neutron resonance measurements for some 200 isotopes. These data are shown in Fig. A.17. These values (Table A.4) have been stored on the evaporation input data tape as the array APRIME (250), which is read immediately after the WAPS array. An input option allows the above method to be used, or a constant value for B_0 can be input.

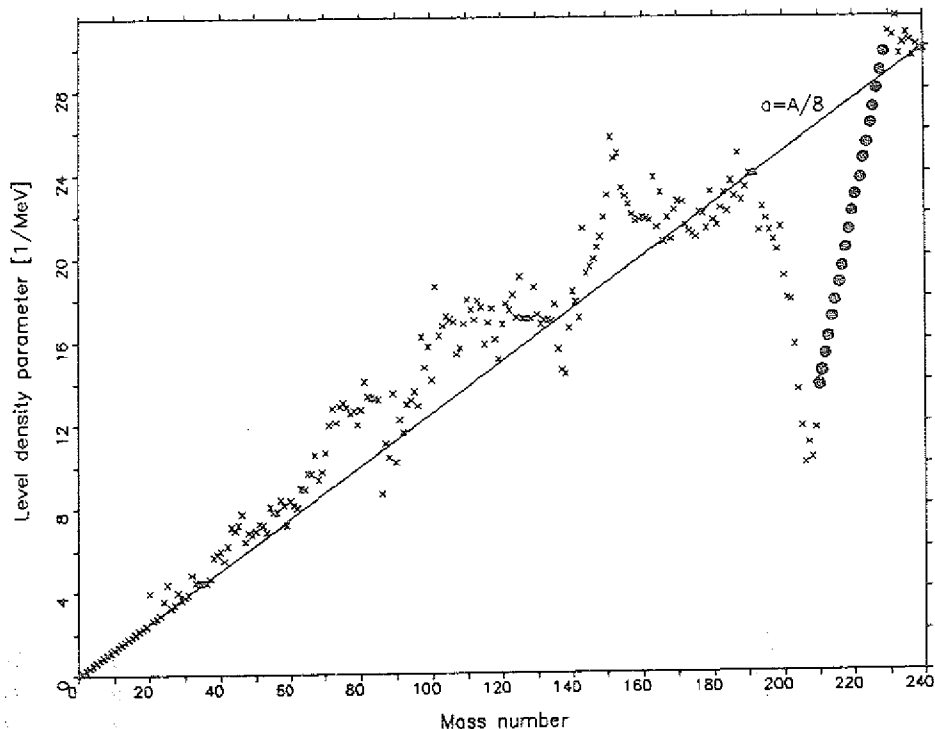


Fig. A.17: Level density parameters deduced by Baba /A.16/ from neutron resonance experiments (x symbols). The $\circ$ symbol denotes our rather arbitrary interpolated values for masses not measured. The linear dependence of $A/8$ is usually assumed in HETC

Table A.4

Level density parameter values used as input for the evaporation model "variable B_0 " option. The integer values are mass numbers and the other numbers are in the corresponding level density parameter values (in MeV^{-1}), as plotted in Fig. A.4

1	0.13	49	6.77	97	16.16	145	19.51	193	21.16
2	0.25	50	6.91	98	14.71	146	19.87	194	22.30
3	0.38	51	7.26	99	15.69	147	20.39	195	21.75
4	0.50	52	7.20	100	14.09	148	20.90	196	21.19
5	0.63	53	6.86	101	18.56	149	21.85	197	20.72
6	0.75	54	8.06	102	16.22	150	22.89	198	20.24
7	0.88	55	7.81	103	16.67	151	25.68	199	21.34
8	1.00	56	7.82	104	17.13	152	24.64	200	19.00
9	1.13	57	8.41	105	17.00	153	24.91	201	17.93
10	1.25	58	8.13	106	16.86	154	23.24	202	17.85
11	1.38	59	7.19	107	15.33	155	22.85	203	15.70
12	1.50	60	8.35	108	15.61	156	22.46	204	13.54
13	1.63	61	8.13	109	16.77	157	21.98	205	11.78
14	1.75	62	8.02	110	17.93	158	21.64	206	10.02
15	1.88	63	8.93	111	17.45	159	21.75	207	10.98
16	2.00	64	8.90	112	16.97	160	21.85	208	10.28
17	2.13	65	9.69	113	17.88	161	21.77	209	11.72
18	2.25	66	9.65	114	17.58	162	21.69	210	13.81
19	2.38	67	10.55	115	15.78	163	23.74	211	14.46
20	3.94	68	9.38	116	16.83	164	21.35	212	15.30
21	2.63	69	9.72	117	17.49	165	23.03	213	16.16
22	2.75	70	10.66	118	16.03	166	20.66	214	16.99
23	2.88	71	11.98	119	15.08	167	21.81	215	17.84
24	3.55	72	12.76	120	16.74	168	20.77	216	18.68
25	4.35	73	12.10	121	17.74	169	22.18	217	19.53
26	3.25	74	12.86	122	17.43	170	22.58	218	20.37
27	3.38	75	13.03	123	18.14	171	22.55	219	21.22
28	3.96	76	12.81	124	17.06	172	21.45	220	22.06
29	3.63	77	12.54	125	19.01	173	21.16	221	22.91
30	3.75	78	12.65	126	17.02	174	21.02	222	23.75
31	3.88	79	12.00	127	17.02	175	20.87	223	24.60
32	4.82	80	12.69	128	17.02	176	22.09	224	25.44
33	4.44	81	14.05	129	18.51	177	22.00	225	26.29
34	4.43	82	13.33	130	17.20	178	21.28	226	27.13
35	4.43	83	13.28	131	16.75	179	23.05	227	27.98
36	4.42	84	13.23	132	16.97	180	21.70	228	28.82
37	4.63	85	13.17	133	16.94	181	21.45	229	29.67
38	5.66	86	8.66	134	16.91	182	22.28	230	30.71
39	5.81	87	11.09	135	17.69	183	23.00	231	30.53
40	5.95	88	10.40	136	15.55	184	22.11	232	31.45
41	5.49	89	13.47	137	14.56	185	23.56	233	29.63
42	6.18	90	10.17	138	14.35	186	22.83	234	30.15
43	7.11	91	12.22	139	16.55	187	24.88	235	30.65
44	6.96	92	11.62	140	18.29	188	22.64	236	30.27
45	7.20	93	12.95	141	17.80	189	23.27	237	29.52
46	7.73	94	13.15	142	17.05	190	23.89	238	30.08
47	6.41	95	13.57	143	21.31	191	23.92	239	29.80
48	6.85	96	12.87	144	19.15	192	23.94	240	29.87

(3) Option for the Treatment of Emitted-Particle Angular Distribution

In the standard HETC code using the EVAP-4 model, emitted evaporation particles are assumed to be isotropic in the laboratory system. There are two effects which can cause the actual distribution to be non-isotropic: (a) for high bombardment energies, the nucleus which evaporates the particles can have rather large angular momentum, and (b) the evaporating nucleus is in movement during particle emission with a certain recoil energy. The dominate effect on emitted angular distribution is the nuclear recoil rather than angular momentum /17/. To indicate the general functional dependence and magnitude of anisotropy, an approximate relationship for the angular distribution (in lab system) of evaporated neutrons is /17/

$$W(\theta) \sim 1 + (v^*/v)\cos\theta$$

where v is the neutron velocity and v^* is the center-of-mass velocity. As an example, for 600 MeV protons incident on a lead nucleus, the angular distribution of 1-MeV neutrons then is $W(\theta)=1+0.23\cos\theta$, so the $W(0^\circ)/W(90^\circ)$ anisotropy is roughly 20% for this case, and larger for lighter nuclei.

Los Alamos has recently modified their version of HETC to take into account the recoil direction of the residual nucleus during evaporation. Evaporated particle directions are selected from an isotropic center-of-mass distribution, and this angle and the recoil direction are transformed to give non-isotropic angular distributions in the laboratory system. These modifications were kindly provided to us by R.E. Prael of Los Alamos, and have been incorporated in the KFA HETC version as an option. This required changes to the following subroutines: MAIN, DATAI, ERUP, RECOIL, DRES, and WTape. If this option is selected, the direction cosines U , V , and W are written on the history tape for each evaporated particle. We have made only a few runs using this option; it seems to work properly but has not been thoroughly tested. It should be noted that this option is available at present only when using EVAP-5; it is not incorporated in the RAL fission model.

(4) Option for High-Energy Fission

The fundamental basis of all of fission models is the statistical model of fission developed by Fong /18/. Basically, the assumption is that the fission process is "slow" (i.e., the nucleus exists in an equilibrium state at any time), so the pro-

bability of a particular fission mode (state of the fission fragments) is proportional to the density of quantum states at the time of splitting. From the Fong theory, the fission mode probability is expressed as a function of eleven variables: $N(A_1, A_2, Z_1, Z_2, C, D, K, E, E_1, j_1, j_2)$, where the subscripts denote the fission fragments; A, Z, and j denote mass number, charge number and angular momentum, and the remaining are energy variables (C, Coulomb; D, deformation; K, translational; E, total; and E_1 , excitation).

The high-energy fission models developed differ in the approximations made in arriving at a practical implementation of the above general expression and in the physical data used. It is not the purpose here to evaluate these various approximations. Indeed, many of the approximations are interdependent, making judgement of the practical importance of particular assumptions difficult. From evaporation theory (e.g., /19/), the probability of particle emission of type i with kinetic energy E is expressed as

$$P_i(E) \propto (2S_i+1) \cdot m_i \cdot E \cdot \sigma_{Ci}(E) \cdot \omega(E^*),$$

with S_i = spin, m_i = mass, and σ_{Ci} = cross section for compound nucleus formation in the inverse reaction. The level density for excitation energy E^* is given by

$$\omega(E^*) = \omega_0 \cdot \exp(2(a \cdot E^* - \delta)^{1/2}),$$

where ω_0 and a are constants for a given nucleus and δ is the pairing energy. The level density parameter, a, is given by

$$a = A/B_0(1+Y(A-2Z)^2/A^2),$$

and $Y \sim 1.5$. From various analyses, B_0 is in the range from about 8 to 20 MeV (e.g., /19/).

The high-energy fission model of the Rutherford and Appleton Laboratories developed by F. Atchison was implemented in the code /20/ (details in Refs. /21,22/). From the intranuclear cascade we get the high energy products plus the residual nucleus which is in a highly excited state. Next the question is whether or not it is undergoing high energy fission. If not, a particle is evaporated (i.e., a neutron, proton etc.) and the same question is asked again for the remaining, still excited nucleus. In this way all steps of de-excitation are treated. On occurrence of fission, two excited nuclei are produced which then decay again by evaporation. In other words, the fission neutrons come from exactly the same processes as the other eva-

poration neutrons, except that the masses of the evaporating nuclei are lower. The problem in implementing a code in HETC which takes fission into account is, of course, that one has to make some decisions, namely whether or not fission is to occur and what will be the scission parameters of the fragments (mass, charge, recoil, excitation).

The Rutherford Appleton Laboratory fission model written by Atchison is based on several general assumptions:

- Fission competes at all stages of nuclear de-excitation, with fission probabilities that vary with the nuclear state.
- Post-fission parameters depend only on the state of the nucleus at the time of fission and not on the previous sequence of events leading to the state of the nucleus at the time of fission.
- Only binary fission occurs. While ternary and high-order fission have been observed (e.g., /23/), they occur with probabilities which are insignificant for the practical applications of interest here.
- Nucleons and charge are conserved between the fissioning nucleus and the two fission fragments. Thus, low-energy neutrons are not emitted at the time of fission, but arise either from the de-excitation of the nucleus before fission following the intranuclear cascade, or from the de-excitation of the fission fragments.

Specific assumptions used in the RAL model to determine the fission probability and post-fission parameters are given below.

(a) Fission Probability

The decision as to whether fission occurs during de-excitation is determined by randomly selecting fission with probability $P_f = (1 + \Gamma_n / \Gamma_f)^{-1}$, where Γ_n and Γ_f are the neutron and fission widths, respectively.

For high-Z nuclei in the actinide region, the fission barrier is relatively constant at about 6 MeV, and P_f is not strongly dependent on excitation energy E^* (e.g., Gavron, et.al. /24/). Therefore, the model assumes $P_f = 0$ for $E^* \leq 6$ MeV and P_f independent of E^* above 6 MeV, with values determined from the expression of Vandenbosch and Huizenga /25/:

$$\log (\Gamma_n / \Gamma_f) = \phi(\hat{Z}) \hat{A} + \psi(\hat{Z})$$

Here $\hat{Z}$ and $\hat{A}$ are the charge and mass numbers of the fissioning nucleus, and ϕ and ψ are tabulated functions which depend only on Z . In the code, the above expression is used for $89 \leq \hat{Z} \leq 99$, with fission not considered for higher $\hat{Z}$.

In the sub-actinide region, the results of experiments show that P_f is strongly dependent on E^* (e.g., Barashenkov, et.al. /26/), and for the RAL model Atchison has made fits to experimental data using statistical model expressions to determine P_f . Separate fits are made for neutron emission and fission to allow separate level density parameters. The expressions for neutron emission are:

$$\Gamma_n = C_1 \cdot (C_2 I_0 + C_3 A^{1/3} I_1 + A^{2/3} (C_4 I_1 + C_5 I_0))$$

$$\begin{aligned} C_1 &= 0.352 & C_2 &= 1.68 & C_3 &= 1.93 \\ C_4 &= 0.76 & C_5 &= -0.05 \end{aligned}$$

$$I_0 = (2a_n)^{-1} ((S_n - 1.0) \exp(S_n) + 1.0)$$

$$I_1 = (8a_n^2)^{-1} ((2S_n^2 - 6S_n + 6) \exp(S_n) + S_n^2 - 6)$$

$$a_n = (A - 1)/8$$

$$S_n = 2(a_n(E^* - B'))^{1/2}$$

$$B' = \text{separation energy} - \text{pairing energy}$$

The expressions used for fission are:

$$\Gamma_f = ((S_f - 1) \exp(S_f) + 1) / a_f$$

$$S_f = 2(a_f(E^* - E_f))^{1/2}$$

$$E_f = C_6 - C_7(Z^2/A) + C_8(Z^2/A)^2$$

$$C_6 = B' + 321.2 \quad C_7 = 16.7 \quad C_8 = 0.218$$

$$a_f/a_n = C_9 + C_{10}((Z^2/A) - C_{11})^2$$

$$C_9 = 1.09 \quad C_{10} = 0.011 \quad C_{11} = 31.09$$

The above expressions are used for $Z \leq 88$. The fit parameters for the fission barrier E_f and the level density ratio, a_f/a_n , are based on data for $Z < 85$, so the model fits are used for extrapolation to obtain the fission probability in the $Z = 85$ to 88 region.

(b) Post-Fission Parameters

At this point, fission has occurred for a nucleus having charge $\hat{Z}$, mass $\hat{A}$, excitation energy $\hat{E}^*$, and recoil energy $\hat{E}_r$, and we desire to determine the corresponding parameters for each of the fission fragments.

The selection of the masses for the fission fragments depends on whether the fission is symmetric (a wide, single peak for the fragment mass distribution) or asymmetric (two peaks). For $\hat{Z}^2/\hat{A} \leq 35$, only symmetric fission is allowed, and the mass A_1 is selected from a Gaussian distribution of mean $A = \hat{A}/2$ and width $\sigma_A = C_{12} + C_{13}\hat{U} + C_{14}\hat{U}^2$, where $C_{12} = 3.97$, $C_{13} = 0.425$, $C_{14} = -2.12 \times 10^{-3}$, and $\hat{U}$ is the excitation above the fission barrier: $\hat{U} = \hat{E}^* - E_f$. This symmetric fission mass distribution is based on the systematics given by Neuzil and Fairhall /27/. If $Z > 88$, the fission energy barrier is determined in the following way, based on the work of Seaborg and Vandenbosch /28/: $E_f = C_{15} + C_{16} \cdot \hat{Z}^2/\hat{A}$, where $C_{16} = -0.36$ and C_{15} has the values 18.8, 18.1, 18.1, and 18.5 for odd-odd, even-odd, odd-even, and even-even nuclei, respectively. For $\hat{Z} < 88$, E_f is calculated using the expression given earlier under the fission probability calculation. With A_1 determined, the mass of the other fission fragment is $A_2 = \hat{A} - A_1$.

If $\hat{Z}^2/\hat{A} > 35$, competition between symmetric and asymmetric fission is allowed, with the asymmetric probability being $P_{asy} = F/(1+F)$ where $F = 4870 \exp(-0.36\hat{E}^*)$. For asymmetric fission, A_1 is selected from a Gaussian of mean $A = 140$ and width $\sigma_A = 6.5$, and then $A_2 = \hat{A} - A_1$.

The charge of the first fragment Z_1 is selected from a Gaussian distribution of mean $\hat{Z}_1$ and width $\sigma_Z = 2$, where $\hat{Z}_1 = (\hat{Z} + Z_1 - Z_2)/2$, $\hat{Z}_1 = C_{15}A_1/(2C_{15} + A_1^{2/3})$, and $C_{15} = 65.5$. Then $Z_2 = \hat{Z} - Z_1$.

As an example for the fission probability computed using the RAL model for various isotopes Fig. A.18 is attached.

The total kinetic energy of the fission fragment in the center-of-mass E_T^i , is selected from a Gaussian distribution having mean $\bar{E}_T = 0.13\hat{Z}^2/\hat{A}^{1/3} - 11.4$ and width $\sigma_E = 0.084E_T^i$. These relations are based on evidence of the strong correlation of fission fragment kinetic energies with the Coulomb repulsion parameter $\hat{Z}^2/\hat{A}^{1/3}$ of the fissioning nucleus (e.g., /23/), and data which suggests that the kinetic energies are narrowly distributed about $\bar{E}_T^i$. The kinetic energy of each fragment, in the center-of-mass, is $E_1^i = A_2E_T^i/\hat{A}$, and $E_2^i = E_T^i - E_1^i$. Isotropic emission in the center-of-mass is then assumed to obtain the kinetic energies E_1 and E_2 in the laboratory system.

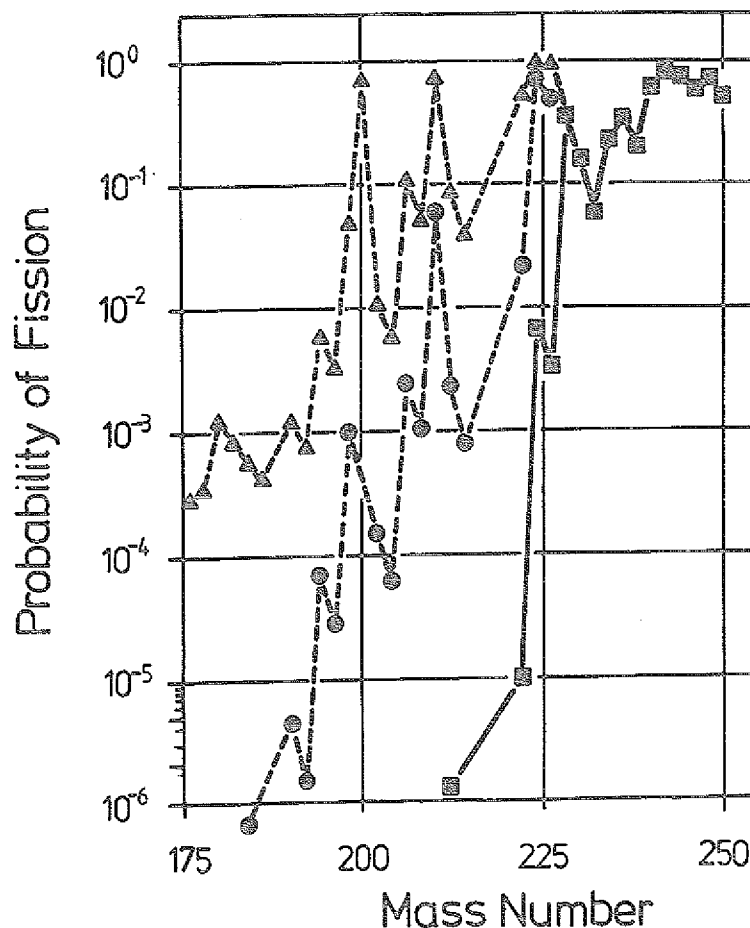


Fig. A.18: Probability of fission using RAL model for various isotopes arbitrarily selected over the range from $A=175-250$. The top, middle, and bottom curves are for assumed excitation energies of 100, 50, and 20 MeV, respectively. (The model assumes that the fission probability is constant for excitation energies above 6 MeV for isotopes having Z above 90)

The excitation energies of the fission fragments is then determined by an energy balance: $E_1^* = (A_1/\hat{A})(\hat{E}^* + B_1 + B_2 - \hat{B} - E_T)$ and $E_2^* = E_1^* A_2/A_1$, where B denotes binding energies.

A.1.3 Thick-Target Transport of Low-Mass Heavy-Ion Beams

The standard version of the HETC code allows protons, neutrons, π^+ , π^- , μ^+ , or μ^- as source particles. Previously, modifications were made (for operation on CDC computes) to treat "light" heavy ion beams having $2 < A < 10$ /29/. The programming for this capability has been converted for operation on IBM computers and incorporated in the HETC-KFA version.

The basic theory is described in Ref. /29/, and only a short summary is given below. The key assumption is that projectile particles can be treated as a cluster of nucleons, and nonelastic interactions computed as the sum of the effects of individual computed using the same intranuclear-cascade evaporation model as used for nucleons and pions. This assumption becomes worse as the A of the projectile increases, because for heavier projectiles a cascade can be induced in the projectile particle as well as in the target nucleus. Thus, the model is expected to be applicable for d, t, He, and alpha beams. The A at which the model becomes invalid is not well defined. The programming for the model allows beams up to A=10.

The basic procedure used when a nonelastic collision occurs is as follows (see also Ref. /29/):

- (a) The energy of each nucleon of the projectile is set to $E_N = (E_i - B_i) / A_i$, where E and B are the kinetic and binding energies of the ion. (These binding energies are set internally for A<4, and must be specified as input for A>4).
- (b) A configuration for the nucleus of the ion is specified.
- (c) Impact parameters and a cascade-evaporation calculation is performed sequentially for each nucleon of the cluster.
- (d) Collisions with hydrogen target nuclei are computed using the same p-p and p-n cross-sections as used by the intranuclear cascade code.
- (e) All atomic processes (ionization energy loss, ranges, etc.) are scaled from relations for protons.

Semi-empirical formulas are used for total ion-nucleus cross-sections. There are no pseudo collisions as in the case of nucleon and pion transport. The additional subroutines for heavy particle transport are indicated in Ref. /29/.

Some results of calculations using this method are given in Ref. /29/ for a few cases: (a) neutron production for 160 MeV deuterons incident on thin targets of various materials, (b) energy deposition in a thick aluminum target for 1-GeV alpha particles, and (c) energy deposition in a thick U-238 target from 330 MeV deuterons. From these cases, the model appears to give reasonably good predictions, but it has not been extensively tested.

A.1.4 Miscellaneous Updates

(1) Corrections of the Charged Particle Slowing Down
Algorithm in Range Calculations

In the former HETC-KFA version the charged particle energy loss is calculated by interpolation from a predefined energy range table.

At the beginning of a flight path $r=R(E)$ gives the range of the particle until cutoff energy will be reached. At the end of a flight path $EC=R^{-1}(r\text{-path})$ gives the remaining energy. If the particle passes from one material to another, a new range is chosen based on the material being entered and the energy at the medium boundary.

Both the range and the final energy are interpolated from the range table which has been setup in 200 logarithmic steps of E . Only the proton range is tabulated, whereas for $\pi^\pm$ and $\mu^\pm$ the range is scaled using the relation

$$R_i(E_i) = (m_i/m_p) \cdot Z_p^2/Z_i^2 \cdot R_p \cdot ((m_p/m_i)E_a)$$

$$\sigma^2(R_i(E_i)) = (m_i/m_p) \cdot \sigma^2 \cdot [R_p(E_p(m_p/m_i))]$$

where m_p , Z_p and R_p are the mass, charge and range-function of a proton, and m_i , Z_i , R_i and E_i are the mass, charge, range-function and energy of any particle of interest. (Ranges are calculated using the code SPAR /37/.)

Both the scaling as described above and the logarithmic interpolation of the range tables (especially in invers direction) caused the amplification of numerical errors of the tabulated data. Furthermore the integration scheme used to calculate the range from dx/dE was rather simple. Thus for applications where small path lengths are frequently used (such as sampling calorimeters) the numerical uncertainties could cause perceptible errors in the energy loss.

Therefore, we have implemented separate range tables for $\pi^\pm$ and $\mu^\pm$ to avoid scaling. We changed the interpolation above 150 MeV to linear interpolation. This seems to fit better the shape of the energy range curve and we avoid the precision loss due to the usage of the log function. In the logarithmic part of the table we stored the logarithm of the range along with range itself to speedup the interpolation routine. At last we changed the integration algorithm and use now the DCADRE function of the IMSL /30/.

In practice these changes have solved the problem so far. Although in future releases of HERMES this procedure will be changed again to implement user selected and tested function setups and offer the slowing down for other particles.

(2) Range Straggling

In some applications involving low-energy charged particle sources in which the fraction of the beam particles that slow down and stop from ionization energy losses rather than undergo nuclear collision is significant, it is necessary to take into account the statistical fluctuations in energy loss. These fluctuations result in a variable range distribution (i.e., range "straggling"). In the standard version of HETC only mean ranges and average energy losses are computed. In the HETC-KFA-2 version energy-loss fluctuations and range straggling are incorporated for the primary beam particles.

The straggled range distribution is taken to be Gaussian about the mean range with a variance given by /31/:

$$\sigma^2 R_i(E \rightarrow E') = 4\pi e^4 Z_i^2 n \int_{E'}^E \frac{(1 - 1/2\beta^2) \cdot K(E)}{(1 - \beta^2) \cdot (1 + (2m_e/m_i)\gamma)} (S_i(E))^{-3} dE$$

where $\sigma^2 R_i(E \rightarrow E')$ = the variance of the range distribution for a particle of type i slowing down from kinetic energy E to E' ,

e = the electron charge,
 Z_i = the charge for the particle slowing down,
 n = the electron density of the stopping medium,
 K = the binding correction factor,
 $\beta^2 = [(E^* + 1)^2 - 1] / (E^* + 1)^2$,
 $E^* = E / m_i c^2$,
 m_e = the electron mass,
 $\gamma = (1 - \beta^2)^{-1/2}$,
 S_i = the stopping power.

The factor K takes into account the binding effect of the atomic electrons at low E .

Ranges are chosen from the distribution

$$p(R_i) dR_i = \frac{1}{\sigma^2 R_i \sqrt{2\pi}} \cdot \exp\left(-\frac{(R_i - R_i)^2}{2\sigma^2 R_i}\right) dR_i$$

where R_i is the mean range.

(3) Low-Energy Transport Cutoffs

An option has been added to allow different cutoff energies for ionization and nuclear collisions for charged particle transport. For protons the input parameter ELOP was used as the energy cutoff for both nuclear collisions and ionization. Now, ELOP is for nuclear collisions only, and a new input parameter, EMIN(1), is used for the proton ionization energy cutoff. New input parameters, EPICUT and EMUCUT, have been added as cutoff energies for charged pions and muons, respectively. Thus, the ionization cutoffs are independent for different charged particles, whereas formerly the pion and muon cutoffs were fixed relative to the proton cutoff specified.

The EMIN array in labeled common /COMON/ now contains the ionization energy cutoffs for charged particles and the neutron collision cutoff. The collision cutoff for protons, ELOP, is now passed through the new common /ECUTP/.

(4) Coulomb Scattering

In the versions of HETC distributed by RSIC there was a problem in that the Coulomb scattering subroutine SPRD did not properly update certain variables at material boundary crossings. This problem was corrected already in HETC/KFA-1 /32/.

(5) Treatment of Pion Decay

Neutral Pion Decay:

Neutral pions (π^0) have a very short half-life and are assumed to decay at the collision site where they are created. Only the source distribution is provided in HETC. The photons from π^0 -decay can be obtained either in EGS4, or in STATIST taking into account the $\gamma\gamma$ - and the Dalitz-decay channels.

Charged Pion Decay:

For charged pions "in flight" (e.g., before coming to rest due to ionization energy loss), both decay and nuclear collisions are taken into account. π^+ -particle which come to rest are assumed to decay or undergo nuclear capture, depending on the setting of the input option (the choice depends upon the density of the target material).

Pion decay in-flight is determined by defining a "macroscopic decay cross section", which analogous to the macroscopic cross section used for nuclear collisions. If the option is specified that π^- -particle become captured when stopped, the capture products are computed using the INC model, with the initial excitation corresponding to the rest-mass energy of 139.6 MeV. The main properties are given in Table A.5.

Table A.5: Properties of pions and muons

(1) Decay Properties

particle	mean lifetime τ_0 [s]	$c\tau_0$ [m]	decay model
π^+	2.6×10^{-8}	7.8	$\pi^+ \rightarrow \mu^+ \quad \nu_\mu$
π^-	2.6×10^{-8}	7.8	$\pi^- \rightarrow \mu^- \quad \nu_\mu$
μ^+	2.2×10^{-6}	~ 660.0	$\mu^+ \rightarrow e^+ \quad \nu_e \quad \nu_\mu$
			$\mu^- \rightarrow e^- \quad \nu_\mu \quad \nu_e$
π^0	0.87×10^{-16}	$\sim 10^{-8}$	$\pi^0 \rightarrow \gamma\gamma$ (98.799%)
			$\pi^0 \rightarrow \gamma e^+ e^-$ (1.198%)

(2) Decay Cross Sections

$$\Sigma_D(E) = 1/\lambda_{\text{decay}} \quad (\text{macroscopic decay cross section})$$

with

$$\lambda_{\text{decay}} = \pi \cdot \tau = \pi \cdot \tau_0 \cdot \gamma \quad (\text{decay mean path})$$

$$(\tau = \gamma \cdot \tau_0, \gamma = [1 - (\pi/c)^2]^{-1/2})$$

The mean decay time in the rest frame is $\tau_0 = 2.6 \times 10^{-8}$ s for pions and 2.2×10^{-6} s for muons. According to Table A.5 the mean-free-path for decay can be written

for pions,

$$L_{\text{decay}}^{\pi} = 7.8 [(E/139.6 + 1)^2 - 1]^{1/2} \quad [\text{m}]$$

and for muons

$$L_{\text{decay}}^{\mu} = 660 [(E/105.7 + 1)^2 - 1]^{1/2} \quad [\text{m}]$$

(The production and transport of neutrinos is not included in the present version of HETC. This extension will be made in a next HERMES release).

The original coding of HETC does not allow charged pion decay to occur explicitly. An option is included to allow analog π -decay. In this context analog decay of charged pions in-flight means, that the decay path length - see above - is sampled from a distribution function using the velocity and the mean lifetime of the pion. If the decay path length is exhausted before reaching a collision point, the pion is discarded and a muon is produced. Weighted decay of charged pions - in the original coding of HETC - means the statistical weight of the pion is reduced along the flight path length by the decay probability during that flight. The weight assigned to the created muon corresponds to the weightloss of the pion.

Weighted decay will improve the statistical error of the produced muons and should be used if mean values are important, whereas analog decay should be used if event-by-event fluctuations will be considered.

A.2 Modification of MORSE-CG

The base program source used is MORSE-CG in Ref. /33/.

A.2.1 Particle source

(1) Fixed point source

NSTRT identical particles are started in one run. The characteristic parameters for one particle, starting point, direction and age are defined on Card D1, starting weight of all particles is given on Card C.

(2) HETC Neutron Source

All neutrons, generated by the present HETC are saved on the HERMES submission file by their parameters, name, energy, direction, location and weight. HETC creates an amount of neutrons varying from batch to batch. MORSE starts neutrons as found on file instead of using NSTRT.

For more detailed information about submission file see 3.4 Structure of Submission Tapes.

A.2.2 Cross-Section Libraries for MORSE

There are three types of cross-sections MORSE accepts:

- (1) Library with ANISN structure, preceded by a special head-record, for instance EPR /34/.
The first three lines of an ANISN structured library are formed by σ_{ab} , $\nu\sigma_f$, σ_{tot} ; cross-section data contain the factor $(2L+1)$.
- (2) ANISN-formatted library, e.g. HILO /2/, AMPX-created data.
Cross section matrix as in (1) preceded by an ANISN-standard format header record.
- (3) Library, created by the program system RSYST, e.g. 53-, 60-, 99-group structured library /35/.

- An RSYST-created library always consists of five head records, containing σ_f , σ_{tr} , σ_{ab} , $\nu\sigma_f$, σ_{tot} , followed by a complete cross-section matrix, that is not multiplied by $(2L+1)$. MORSE ignores the first 2 head records and multiplies the matrix data by $(2L+1)$.
The position of σ_{g-g} that separates the upscatter matrix (line 6 to IHS-1) from the downscatter matrix (IHS to ITBL) is given by IHS; no structure with IHS=0 can be used in MORSE.
(See input description MORSE-CG, Section 3.3).

A.2.3 Changes of Partly Analogous Simulation in MORSE

MORSE does not use analog collision kernels. This means that no decision is made on the collision type. Each collision is taken to represent elastic scattering, inelastic scattering, absorption, fission and n,γ process at a time. The final state of a collision is computed as a compound of the final states of the distinct collision types.

This means, the statistical weight of the incident particle is multiplied by the non-absorption probability including the multiplicity of n,xn collisions. The final group and direction is selected from the cross-section set which also represents the superposition of elastic and inelastic cross sections.

Fission neutrons are not included in the scatter matrix of the cross section because those would dominate the spectrum. Thus, fission neutrons are generated as secondary particles. Their statistical weight was derived from the differential fission probability table. The amount of fission particles used was taken from the FWLO array of the MORSE input.

γ 's have been generated in a similar way; the γ -weight was set to the generation probability $\nu_\gamma * p(n, \gamma)$ and the number of γ 's was taken from the GWLO (the γ weight table).

This method of simulated Monte-Carlo results in correct mean value estimates of flux and collision rates. On the other hand it will not create physical particle histories rather than statistical representatives of classes of histories having the same physical properties. Fluctuations are normally reduced, speeding up the code.

However, if fluctuations should be measured themselves one has to use a higher degree of analogism with the physical behaviour of particle histories. Therefore, we have changed the algorithm of γ generation and fission neutron generation using $\nu * p_{fiss}$ and $\nu * p_{n, \gamma}$ to find the weight of secondaries to be generated. Then we divide this weight into portions of FWLO or GWLO, respectively. Setting FWLO and GWLO to one will then result in the generation of the physically correct amount of fission neutrons and γ 's. Setting the russian roulette weight and the splitting weight to one will have the same effect for absorption and n, xn scattering.

Although, these changes adjust the number of secondary particles with respect to the distinct collision type, the collision remains a superposition of different possible types. This problem cannot be solved without separating the cross-sections into the various collision channels which in effect would mean, that we have to use a completely new Monte-Carlo program with completely new cross-section data.

A.2.4 Available Estimators

Six different estimators are available in the MORSE release:

- Volume detector: volume-flux and volume-light
- point detector
- Surface detector: surface flux, surface current and surface energy current
- Volume detectors:

(1) Volume flux detector

The volume flux in energy group E is calculated by

$$\phi(E)_{Vol} = f/NM \cdot \sum_{j=1}^M \sum_{i=1}^{N_j} (WT_i * ETATH_i)$$

with f global normalization factor
 NM number of source particles started in all batches
 M number of batches
 N_j number of events in batch j
 WT_i weight of neutron i before collision
 $ETATH_i$ distance neutron i has to travel within the same medium (usually called as track length)
 Vol Volume

There is the probability to estimate energy-, time-, time- and energy-, or angle- and energy dependent values, with a normalization to ΔE , Δt , and $\Delta \Omega$. To get a certain response

$$R_V = \sum_{g=1}^{NE} \phi(E) Vol \cdot Res_g$$

the user has to specify response functions and multiply them with the determined flux $\phi(E) Vol$.

(2) Volume Light Detector

The volume light detector is used to measure scintillation light induced by neutron scattering. Only scattering with 1H is taken into account. Thus the detector measures the light generated by recoil protons. At each scattering point the detector decides whether the neutrons were scattered at 1H or not, using the differential hydrogen scattering probability

$$P_{1H} = \frac{\sigma_{sc}(1H) * N_{1H}(med)}{\Sigma_{tot}(med)}$$

(med = medium in which scattering takes place.)

The neutron is assumed to generate a recoil proton with energy uniformly distributed between zero and the neutron energy. The generated proton is assumed to be stopped within the scintillator.

Using Birk's law /36/ we can calculate the light response of the proton using ranges calculated via the SPAR code /37/

$$L_p(E_p) = \int_0^{E_p} \frac{dL_p}{dE_p} dE = \int_0^{E_p} \frac{S^*}{1 + kB(dE/dx)_p} dE$$

where S is the scintillation efficiency and kB a quenching factor

Folding $L_p(E_p)$ with the distribution function of E_p for a given neutron energy E_n will result in the mean scintillation effect of the neutron

$$L_n(E_n) = \frac{\int_0^{E_n} L_p(E_p) dE_p}{\int_0^{E_n} dE_p}$$

$L_n(E_n)$ is calculated for each neutron group boundary. For simplicity $L_n(E_n)$ is considered to be linear in E for a given energy group. Thus for group g we can use

$$L_g = (L_n(E_g) + L_n(E_{g+1})) / 2$$

as the mean light response of a neutron in group g. Consequently, the visible energy induced by neutrons of energy group g is calculated as

$$E_{Lg} = f / NM \sum_{j=1}^M \sum_{i=1}^{N_j} WT_i * L_g$$

with

- f = global normalization factor
- NM = number of source particles started in all batches
- M = number of batches
- N_j = number of events in batch j
- WT_i = weight of neutron i before collision
- L_g = light response as described above

To get the volume light response over the total range

$$R_L = \sum_{g=1}^{NE} (E_{Lg} * Rres_g)$$

the user has to specify response functions res_g . An example of the light response for neutrons is given in Appendix E.

- Point Detector

The point detector calculations performed by MORSE is using the "next-event" estimator which scores, from each collision point (collided) or source point (uncollided), the probability of the next event being at the dx x in energy group g is calculated as

$$\phi(E,r) = f/NM \sum_{j=1}^M \sum_{i=1}^{N_j} \frac{(WT_i)_j e^{-\mu_{ij} P_{ij}}}{(x_{ij}-x_p)^2 + (y_{ij}-y_p)^2 + (z_{ij}-z_p)^2}$$

with $(WT_i)_j$ = weight of the particle leaving event i in batch j
 μ_{ij} = number of mean free path between event i and detector
 P_{ij} = probability, per steradian, for a particle to be scattered into the detector direction
= in case of uncollided flux calculation, P_{ij} is assumed as $1/4 \pi$
 x_p, y_p, z_p = coordinates of detector point
 x_{ij}, y_{ij}, z_{ij} = coordinates of event point
 NM = number of source particles started in all batches
 M = number of batches
 N_j = number of events in batch j
 f = global normalisation factor
 r = coordinates of the detector

MORSE outputs for all detectors the energy dependent flux as well as the uncollided and the total response using response functions res_g

$$R_p = \sum_{g=1}^{NE} (\phi(E,r) \cdot res_g)$$

More detail informations about the subroutines SDATA, RELCOL and EUCLID is given in the MORSE manual /33/.

Calculations with point detector are extremely time consuming. To diminish time, the user should specify regions, that are valid for the estimation, i.e., collisions, which do not occur inside of these regions, will be disregarded. It has to be emphasized that the user should exclude only such regions which will not significantly contribute to the flux at the detector.

For detail information about additional input for point detector see Section 3.3 MORSE Input Description.

- Surface Detector

(1) Surface Flux Detector

Surface flux in energy group g is calculated by

$$\phi(E)_{\text{Surf}} = f/NM \cdot \sum_{j=1}^M \sum_{i=1}^{N_j} (WT_i / \text{abs}(\cos_i))_g$$

with f = global normalisation factor
 NM = number of source particles started in all batches
 M = number of batches
 N_j = number of events in batch j
 $\cos_i$ = cosine of angle between direction of flight and the normal to the passed surface
 WT_i = weight of neutron before collision
 Surf = surface of the detector

The surface is defined by its regions on both sides. All particle passing this surface are counted disregarding direction of flight. The flux is not normalized to the surface. Total flux is calculated in cases of response function input.

(2) Surface Current Detector

the surface current is computed as

$$J_g = f/NM \cdot \sum_{j=1}^M \sum_{i=1}^{N_j} (WT_i)_g$$

(3) Surface Energy Current Detector

The surface energy current is computed as

$$J = f/NM \cdot \sum_{j=1}^M \sum_{i=1}^{N_j} (WT_i * E_i)_g$$

with E_i energy of neutron i uniformly distributed between E_g and E_{g+1} .

The surface response for all surface detectors is calculated, if response functions Res_g are specified.

A.3 Modifications of EGS4

The EGS4 code is from the physical point of view adopted to the HERMES system as it is. Only the implementation, input-output- and the geometry interface is changed. This is described above in Section 3.4. It is assumed here that the user is familiar with EGS4 physics, otherwise he should read the EGS4 user's manual /38/ and associated literature /39-40/ where a very detailed description of the physics models is given.

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APPENDIX B
DESCRIPTION OF INPUT DATA USED BY THE EVAPORATION
MODEL IN THE HETC CODE

B.1 Theory and Historical Background

The basic evaporation model theory used in the original HETC code /1/ is that of Weisskopf /2/, with much of the method of implementation based in the work of Dostrovsky, et al. /3-5/. The evaporation model used in HETC is an evolution of an evaporation code called EVAP, originally written by Dresner /6/ with a series of revisions by Guthrie /7, 8/; the nature of these revisions are indicated in TABLE B.1.

Table B.1
Basic Differences in EVAP Series of Evaporation
Model Codes

Evaporation Code	Comments
1 EVAP /6/	original code
2 EVAP-2 /7/	only 6 types of evaporated particles considered instead of 19 in EVAP updated nuclear masses allows ^8Be as a residual nucleus to split immediately into two α particles recycles (up to a max. of 10 times) if get kinetic energy selected greater than allowed by excitation energy
3 EVAP-3 /7/	like EVAP-2 except effects of kinetic energies of recoil nuclei taken into account
4 EVAP-4 /8/	identical to EVAP-3 except for "improved-termination" scheme for particle emission

These "stand-alone" EVAP codes have been incorporated as subroutines (and used in conjunction with the cascade model) to treat high-energy nuclear collisions in the thick-target radiation transport code HETC, and its predecessors NMTC and NTC, as indi-

cated in TABLE B.2. Various updates have been made for the nuclear mass data used, and this is also indicated in TABLE B.2.

Table B.2
Overview of Nuclear Data Input Used in the EVAP
Series of Evaporation Model Codes

Evaporation Code	Transport Code Used in	Nuclear Mass Data Used
1 EVAP	NTC /9/	(a) from tables compiled by Wapstra (1955) /11/ and Huizenga (1955) /12/ for bases where measured masses exist (b) semi-empirical formula of Cameron (1957) /13/ for nuclei not measured
2 EVAP-2 and EVAP-3	NMTC /10/	(a) Mattauch, et al. (1965) /14/ tabulation of binding energies (b) Cameron semi-empirical mass formula /13/ for nuclei not tabulated by Mattauch et al. (c) "shell-plus-pairing" energy corrections for Z or $N < 13$ from Peele and Aebersold /15/ rather than from Cameron
3 EVAP-4	HETC /1/ (RSIC Version)	same nuclear data as EVAP-2 and EVAP-3
4 EVAP-4' (unpublished)	HETC (some versions)	updated nuclear masses from Wapstra and Gove (1971) /16/

The evaporation data input described in this paper corresponds to that used by EVAP-4, which is the form of the evaporation model used in the version of HETC distributed by the Radiation Shielding Information Center (RSIC).

B.2 Comments on Computational Method

Some of the input data for the evaporation model consists of pre-computed functions which are used in various steps of the calculation. To define the meaning of these, and other input data, we briefly summarize here some of the steps of the evaporation calculation.

The probability $P_i(E)$ that a nucleus excited to energy U will emit a particle of type i having kinetic energy E is assumed to be expressed as

$$P_i(E) \propto g_i \cdot m_i \cdot E \cdot \sigma_{ci}(E) \cdot \omega(E^*), \quad (B.1)$$

where g_i is the number of spin states, m_i is the emitted particle mass, σ_{ci} is the inverse cross section (i.e., the compound nucleus formation cross section corresponding to bombarding the residual nucleus with particles of type i and energy E), E^* is the excitation energy of residual nucleus ($E^* = U - Q - E$, where Q is the binding energy of particle i), and ω is the density of energy levels of the residual nucleus.

The form for ω used is

$$\omega(E^*) \propto \exp[2 \cdot [a \cdot (E^* - \delta)]^{1/2}] \quad (B.2)$$

where δ is the pairing energy and

$$a = A/B_0 \cdot (1 + Y\Delta^2/A^2),$$

with A =mass number, B_0 =8 MeV, Y =1.5, Δ = $A-2Z$, and Z =charge number. (Since Eqs. (B.1) and (B.2) are always involved in ratios in the computations, constants have been omitted.) (see updates in Appendix A, Section A.1.2).

The inverse cross section is taken to be the geometric cross section modified by an empirical expression to take into account energy dependence, and a Coulomb barrier correction for charged particle emission. For neutrons,

$$\sigma_{cn}(E) = \pi \cdot R^2 \cdot \alpha(1 + \beta/\alpha) \quad (B.3.1)$$

$$\alpha = 0.76 + 1.93 \cdot A^{-1/3} \quad (B.3.2)$$

$$\alpha\beta = 1.66 \cdot A^{-2/3} - 0.05 \quad (B.3.3)$$

$$R = 1.7 \cdot A^{1/3} \quad (B.3.4)$$

with R in units of fermions ($1 \text{ fm} = 10^{-13} \text{ cm}$). For charged particle emission,

$$\sigma_{ci}(E) = \pi \cdot R^2 \cdot (1 + C_i) \cdot (1 - k_i V_i / E),$$

for $E > k_i \cdot V_i$, and zero otherwise. Here V_i is the Coulomb barrier calculated from the classical electrostatic expression and the factor k_i is inserted to take into account in an approximate way barrier penetration effects. The numerical values used for C_i and k_i are mainly those chosen by Dostrovsky, et al. /4/ to give a good fit to continuum-theory cross sections. Values for protons and alpha particles are given in Table B.3, and the others are determined from the following relations:

$$k_d = k_p + 0.06 \quad C_d = C_p / 2$$

$$k_t = k_p + 0.12 \quad C_{H-3} = C_p / 3$$

$$k_t = k_\alpha - 0.06 \quad C_t = C_\alpha = 0$$

The expression used for Coulomb barrier is

$$V_i = Z_i \cdot Z \cdot e^2 / (R + R_i) \quad (\text{B.4})$$

where e is the electron charge and R_i is zero for protons and 1.2 fm for all other particle types.

Table B.3
Parameters Used for Computing Inverse
Charged-Particle Cross Sections

Z	K_p	C_p	k_α
10	0.36	0.08	0.77
20	0.51	0.00	0.81
30	0.60	-0.06	0.85
40	0.66	-0.10	0.89
50	0.68	-0.10	0.93
60	0.69	-0.10	0.97
>70	0.69	-0.10	1.00

The first step in computing particle emission from an initial nucleus having A , Z , and U is to determine the total (over all energy) emission probability for each particle type which is

$$P_i = P_i / \left(\sum_{i=1}^6 P_i \right), \text{ with}$$

$$P_i = g_i \cdot m_i \cdot \int E \cdot \sigma_{ci}(E) \cdot \omega \cdot (U - Q_i - \delta - E) dE$$

where the upper limit of the integration is $U - Q_i - \delta$ and the lower limit is $k_i \cdot V_i$. (This equation is applicable, of course, only if emission is energetically possible -- i.e., if $U > k_i \cdot V_i + Q_i + \delta$. Otherwise, $P_i = 0$).

Based on the work of Dostrovsky, et al. /3/, this integral is performed analytically. The result (within a common numerical factor) is:

for neutrons,

$$P_i = A^{2/3} \cdot \alpha \cdot (I_1(S) + \beta \cdot I_0(S)) \cdot e^S, \quad (B.5.1)$$

and for charged particles,

$$P_i = (g_i \cdot m_i / 2) \cdot (1 + C_i) A^{2/3} \cdot I_1(S) \cdot e^S \quad (B.5.2)$$

where

$$S = 2 \cdot (a \cdot (U - k_i V_i - Q_i - \delta))^{1/3} \quad (B.5.3)$$

$$I_0(S) = (2a)^{-1} (S - 1 + e^{-S}) \quad (B.5.4)$$

$$I_1(S) = (8a^2)^{-1} (2S^2 - 6S + 6 + e^{-S} \cdot (S^2 - 6)) \quad (B.5.5)$$

The kinetic energy selection for the evaporated particle is based on the equation

$$N(E_i) dE_i = T_i^{-2} \cdot W_i \cdot \exp(-W_i/T_i) dE_i$$

where $W_i = E_i - k_i V_i$ and $T_i = 1/2 \langle W_i \rangle$. The Monte Carlo selection of E_i is realized by setting

$$E_i = 2T_i \cdot X + k_i V_i$$

where X is selected from the distribution $(x \cdot \exp(-x))$, which is equivalent to setting X equal to one-half the sum of two random numbers on the interval $(0,1)$. The expression for T_i , is, for neutrons,

$$T_i = 1/2 \cdot [I_2(S) + \beta \cdot I_1(S)] / [I_1(S) + \beta \cdot I_0(S)] \quad (B.6.1)$$

and for charged particles,

$$T_i = 1/2 \cdot I_2(S) / I_1(S), \quad \text{where} \quad (B.6.2)$$

$$I_2(S) = (32a^3)^{-1} (8S^3 - 48S^2 + 120S - 120 + e^{-S} \cdot [S^4 - 12S^2 + 120]) \quad (B.6.3)$$

After a particle is evaporated, the procedure is repeated with updated nucleus parameters:

$$A' = A - A_i; \quad Z' = Z - Z_i; \quad U' = U - Q_i - E_i$$

B.3 Description of Evaporation Model Input Data

The HETC code requires an input data tape in which the first part consists of data used by the intra-nuclear-cascade model, and the second consists of data used in the evaporation model calculation. The input data arrays for the evaporation model are listed in TABLE B.4 (in the order in which they appear on the data tape), and are discussed below.

The arrays P0, P1 and P2 are pre-computed functions which are used in evaluating the S dependence of the functions I_p , I_1 , and I_2 (Eqs. (B.5.4), (B.5.5), and (B.6.3)), and are defined as follows:

$$P0 = 1/2 \cdot [(S-1)\exp(S)+1] \exp(-50)$$

$$P1 = 1/8 \cdot [(2S^2-6S+6)\exp(S)+S^2-6] \exp(-50)$$

$$P2 = (4/32) \cdot [8S^3-48S^2+120S-120+\exp(-S) \cdot$$

$$\cdot (S^4-12S^2+120)] / [2S^2-6S+6+(S^2-1)\exp(-S)]$$

The values stored on tape have been computed for $S=0.0$ through $S=100.0$ in steps of 0.1, for a total of 1001 values for each array. Over this S range, the expressions in the brackets for P0 and P1 can vary over many orders of magnitude; hence, an arbitrary normalization factor of $\exp(-50)$ is included.

For all variables dependent on the evaporation particles (Items 4 through 8 in Table B.4) the order assumed is neutrons, protons, deuterons, tritons, helium-3, and alpha particles. Thus, $IA=1, 1, 2, 3, 3, 4$, and $IZ=0, 1, 1, 1, 2, 2$.

The RHO array contains values for the radii of evaporated particles (corresponding to R_i of Eq. (B.4)) in units of 1.7 fermions. Thus, $RHO(J)=0.0$ for $J=1$ or 2, and $RHO(J)=0.70588$ for $J=3$ through 6.

Table B.4

Definition of Input Data for HETC Evaporation Model

Array	Dimension	Definition
1 P0	1001	pre-computed functions
2 P1	1001	used in evaluating
3 P2	1001	I_0 , I_1 , and I_2 (see text)
4 IA	6	mass numbers of possible evaporation particles
5 IZ	6	charge number of possible evaporation particles
6 RHO	6	radius of evaporation particles
7 OMEGA	6	(not used)
8 EXMASS	6	mass excess values (in MeV) of evaporation particles
9 CAM2	130	shell-plus-pairing energies for nuclei having charge number 1 through 130
10 CAM3	200	shell-plus-pairing energies for nuclei having neutron number 1 through 200
11 CAM4	130	pairing energies for nuclei having charge numbers 1 through 130
12 CAM5	200	pairing energies for nuclei having neutron numbers 1 through 200
13 T	3,7	constants for computing inverse cross sections (see text)
14 RMASS	297	$RMASS = A^{1/3}$, for $A = 1$ through 295
15 ALPH	297	$ALPH = 0.76 + 1.93 A^{-1/3}$ for $A = 1$ through 295
16 BET	297	$BET = (1.66A^{-2/3} - 0.05)/ALPH$, for $A = 1$ through 295
17 WAPS	250,20	atomic mass data in terms of mass excess values (in MeV); WAPS(I,J) corresponds to a nuclide with mass number $I=A$, J =isobar index (see text)

The EXMASS array contains mass excess values in MeV units for the evaporated particles, which are used in computing the Q values of Eq. (B.5.3) in the following way. Let $M(Z,A)$ denote the mass of an excited nucleus which evaporates a particle of mass m , leaving a residual nucleus of mass $M'(Z',A')$, so

$$Q = M'(Z',A') + m - M(Z,A)$$

In terms of mass excess, $M(Z,A) = A + \Delta M(Z,A)$, etc., where ΔM denotes the mass excess. Therefore, Q can be written as

$$Q = \Delta M'(Z',A') - \Delta M(Z,A) + \Delta m$$

where $\Delta m = \text{EXMASS}$ is the mass excess of the emitted particles. The numerical values stored are from the binding energies given in the 1964 Mass Tables /B.14/, converted to mass excess values as described in Ref. /7/.

The CAM array contain shell and pairing energy corrections, which are terms in the semi-empirical atomic mass formula of Cameron /13/. This formula is used to compute the mass excess of all nuclei for which data are not available from mass tables (as discussed later in connection with the WAPS array). The formula used is that given in Eq. (B.1) of Cameron's 1957 paper /13/:

$$\Delta M(A,Z) = [8.367A - 0.783Z + E_v + E_s + E_c + E_x] + S(Z,N) + P(Z,N)$$

Expressions for the terms E_v (volume energy), E_s (surface energy), E_c (Coulomb energy), and E_x (Coulomb exchange energy), which are functions of only Z and A , are given in Cameron's paper. In HETC, the quantity above in brackets corresponds to FUNCTION CAM(A,Z). The terms S and P are empirically determined corrections to the reference mass formula to take into account shell effects and "pairing" energie (i.e., the extra stability of nuclei with "paired" neutrons and protons). It is assumed that S and P can be written as independent functions of the neutron ($N=A-Z$) and proton numbers:

$$S(Z,N) = S(Z) + S(N); \quad P(Z,N) = P(Z) + P(N)$$

The input data arrays correspond to:

$$\text{CAM2} = S(Z) + P(Z); \quad \text{CAM3} = S(N) + P(N)$$

$$\text{CAM4} = P(Z); \quad \text{CAM5} = P(N)$$

In the code, these arrays are used in the mass formula,

$$\Delta M(Z,A) = \text{ENERGY}(A,Z) \quad (\text{code variable name})$$

$$= \text{CAM}(A,Z) + \text{CAM2}(Z) + \text{CAM3}(A-Z)$$

and for the pairing energies alone (e.g., Eq. (B.5.3)),

$$\delta = P(Z) + P(N)$$

or in the code variable names,

$$\text{CORR} = \text{CAM4}(Z) + \text{CAM5}(A-Z).$$

The numerical input values for CAM2(Z) and CAM3(N) for Z=13 through 130 and N=13 through 200 are those given in Table 1 of Cameron's 1957 paper. For Z=1 through 12 and N=1 through 12, the values given in Table 2 of Peele and Aebersold (15/ are used. For CAM4(Z) and CAM5(N), the values for Z=13 through 130 and N=13 through 200 correspond to (the negative of) the values in Table 1 of Cameron's 1958 paper, and the values for lower Z and N are (the negative of) the values given in Table 4 of Peele and Aebersold.

The T array contains values for the parameters used in computing inverse cross sections. The numerical values in this array are the same as given previously in Table B.3, with $T(1,J)=k_p$, $T(2,J)=k_\alpha$, and $T(3,J)=C_p$. The index $J=1, 7$ denotes the values evaluated at the seven charge number intervals given in Table B.3.

The next three arrays contain values for A-dependent functions evaluated at A=1 through 295. RMASS is used in computing the target nucleus radius, Eq. (B.3.4), and ALPH and BET correspond to α and β as defined in Eqs. (B.3.2) and (B.3.3). It should be noted that while these arrays are dimensioned 297, the values on tape for array locations 296 and 297 are not meaningful.

The WAPS array contains atomic mass data in terms of mass excess values in MeV units. WAPS(I,J) correspond to a nuclide with mass number $I=A$ and J is an isobar index, which is defined as

$$J = A - 2Z - J' + 10, \quad \text{with}$$

$$J' = (A - 1)/(1 + 124/A^{2/3})$$

where J and J' are integers.

For the data tape used with the EVAP-4 version of the evaporation model, the atomic mass data on tape are taken from the 1964 Atomic Mass Table /B.14/. For locations in the WAPS array where values are not available from the 1964 tables, they have been generated and stored in the WAPS array using the Cameron formula /13/. It should be noted that while the WAPS array is dimensioned 20 for each A, the J and J' algorithms for storage and retrieval are such that only 10 values are meaningful for each A. Values needed for A and Z outside of this range are computed by the code using the Cameron formula.

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APPENDIX C

PROBLEM INDEPENDENT INPUT TO THE CASCADE MODEL

C.1 Introduction

HETC needs problem independent data for the intranuclear cascade- and evaporation model calculations. The purpose here is to describe the input data used by the intranuclear cascade model. (The sources of the data described here are not discussed; they have been given by Bertini /1/.)

The nuclear-data input tape contains 5 records of information used by the intranuclear cascade model, followed by 13 records of data pertaining to the evaporation model (Table C.1). In this chapter, we define these first 5 records; the evaporation model data arrays are defined in Appendix B.

Table 1. Nuclear model data arrays used as input to the HETC code. Records 1 - 5 are for the intranuclear cascade model, Records 6 - 18 for the evaporation model.

Record No.	No. Words	Arrays
1	600	CRSC(1)...CRSC(600)
2	600	CRSC(601)...CRSC(1200)
3	600	CRSC(1201)...CRSC(1800)
4	600	CRSC(1801)...CRSC(2400)
5	29849	DCLN(80), DCIN(115), PPAC(19), POAC(19), FMXSN(161), FMXDN(130), FMXSP(117), PDCI(60), PDCH(55), DCHN(143), DCHNA(36), DCHNB(60), PSPCL(158), PDPCL(130), SPCLN(158), DPCLN(130), FSLN(176), FRINN(161), DMIN(101), PPSCL(117), PNSCL(117), PMSCL(117), PNNSL(117), PCFSL(234), FRIPN(117), PNMI(101), PNFSL(234), PNEC(126), PNNEC(126), PMXC(126), PMEC(126), PPEC(126), PEC(176), ECN(176), PPDC(6426), PMDD(6426), PMDX(6426), PNDD(6426)
6	3003	PO(1), P1(1), P2(1)...PO(1001), P1(1001), P2(1001)
7	12	IA(1)...IA(6), IZ(1)...IZ(6)
8	12	RHO(1)...RHO(6), OMEGA(1)...OMEGA(6)
9	6	EXMASS(1)...EXMASS(6)
10	130	CAM2(1)...CAM2(130)
11	200	CAM3(1)...CAM3(200)
12	130	CAM4(1)...CAM4(130)
13	200	CAM5(1)...CAM5(200)
14	21	((T(I,J),J=1,7), I=1,3)
15	297	RMASS(1)...RMASS(297)
16	297	ALPH(1)...ALPH(297)
17	297	BET(1)...BET(297)
18	5000	((WAPS(I,J),I=1,250), J=1,20)

In Section C.2 the input data arrays are defined, and some are displayed graphically in Section C.3.

C.2 Definition of Input Arrays

A brief description of each input data array is given in Table C.2 and some further comments are given below.

Table 2. Definition of input data arrays for intranuclear cascade model.

Array No.	Array Name	No Words	Description	Energy Range (MeV)	ΔE (MeV) for Tabulation
1.	CRSC	600	CRSC (1) through CRSC (600) contains the following nuclear data for $A = 1$ through 60: (a) mass number, A ; (b) radial boundaries (3) defining nuclear regions, (c) nucleon density per nucleon for each region, and (d) Fermi energy per nucleon for each region.	---	---
2.	CRSC	600	CRSC (601) through CRSC (1200) contains nuclear data like Record 1 for $A = 61$ through 120.	---	---
3.	CRSC	600	CRSC (1201) through CRSC (1800) contains nuclear data like Record 1 for $A = 121$ through 180.	---	---
4.	CRSC	600	CRSC (1801) through CRSC (2400) contains nuclear data like Record 1 for $A = 181$ through 239. (Data included for $A = 240$ to fill record, but not meaningful.)	---	---
5.	DCLN	80	neutron-proton differential cross section (energy dependence of coefficients for angular distributions)	0 - 300	20
6.	DCIN	115	same meaning as Record 5	300 - 740	20
7.	PPAC	19	τ^+ - p absorption cross sections (in cm^2)	0 - 360	20

Table 2. Definition of input data arrays for intranuclear cascade model (con't)

Array No.	Array Name	No. Words	Description	Energy Range (MeV)	ΔE (MeV) for Tabulation
8.	POAC	19	τ^+ - p absorption cross section (in cm^2)	0 - 360	20
9.	FMXSN	161	maximum value of pion mass distribution vs. relative kinetic energies for nucleon-nucleon single production reactions	300 - 2500	20
10.	FMXDN	130	pion mass sampling distribution for nucleon-nucleon double pion production	920 - 3500	20
11.	FMXSP	117	like record 9 but for pion-nucleon single pion production	180 - 2500	20
12.	PDCI	60	proton-proton differential cross sections, tabulated at 440, 590, 650, 800, and 1000 MeV	440 - 1000	--
13.	PDCN	55	proton-proton differential cross sections, tabulated at 1000, 1400, 2170, 2900, and 4400 MeV	1000 - 4400	--
14.	DCIN	143	per cent scattering in backward direction for neutron-proton differential cross sections	660 - 3500	20
15.	DCINA	36	neutron-proton differential cross sections in forward direction tabulated at 630, 1540, 2560, and 3520 MeV	630 - 3520	--

Table 2. Definition of input data arrays for intranuclear cascade model (con't)

Array No.	Array Name	No. Words	Description	Energy Range (MeV)	$\Delta E(\text{MeV})$ for Tabulation
16.	DCHNB	60	neutron-proton differential cross sections in backward directions, tabulated at 630, 2040, 2290, 2850, and 3500 MeV	630 - 3500	--
17.	PSPCL	158	proton-proton single pion production cross sections (in cm^2)	360 - 3500	20
18.	PDPCCL	130	proton-proton double pion production cross sections (in cm^2)	920 - 3500	20
19.	SPCLN	158	neutron-proton single pion production cross sections (in cm^2)	360 - 3500	20
20.	DPCLN	130	neutron-proton double pion production cross sections (in cm^2)	920 - 3500	20
21.	FSIN	176	probability of producing two isobars in nucleon-proton double pion production	0 - 3500	20
22.	FRINN	161	tabulated values of integrals for isobar sampling as a function of nucleon-nucleon relative kinetic energies	300 - 3500	20
23.	DMIN	101	tabulated values of integrals for isobar sampling for nucleon-nucleon collisions as a function of the variable R, for R values from 0.0 to 1.0 in steps of 0.01	--	--

Table 2. Definition of input data arrays for intranuclear cascade model. (con't)

Array No.	Array Name	No. Words	Description	Energy Range (MeV)	$\Delta E(\text{MeV})$ for Tabulation
24.	PPSCL	117	π^+ - proton single pion production cross sections (in cm^2)	180 - 2500	20
25.	PNSCL	117	π^0 - proton single pion production cross sections (in cm^2)	180 - 2500	20
26.	PMSCL	117	π^- - proton single pion production cross sections (in cm^2)	180 - 2500	20
27.	PNNSL	117	π^0 - neutron single pion production cross section (in cm^2)	180 - 2500	20
28.	PCFSL	234	final states for π^- - proton single production	180 - 2500	20
29.	FRIPN	117	like Record 22 but for pion-nucleon collisions	180 - 2500	20
30.	PNMI	101	like Record 23 but for pion-nucleon collisions	--	--
31.	PNFSL	234	final states for π^0 - proton single pion production	180 - 2500	20
32.	PNEC	126	π^0 - proton elastic scattering cross sections (in cm^2)	0 - 2500	20
33.	PNNEC	126	π^0 - neutron elastic scattering cross sections (in cm^2)	0 - 2500	20

Table 2. Definition of input data arrays for intranuclear cascade model. (con't)

Array No.	Array Name	No. Words	Description	Energy Range (MeV)	ΔE (MeV) for Tabulation
34.	PMXC	126	π^- - proton charge exchange cross sections (in cm^2)	0 - 2500	20
35.	PMEC	126	π^- - proton elastic scattering cross sections (in cm^2)	0 - 2500	20
36.	PPEC	126	π^+ - proton elastic scattering cross sections (in cm^2)	0 - 2500	20
37.	PEC	176	proton-proton elastic scattering cross sections (in cm^2)	0 - 3500	20
38.	ECN	176	neutron-proton elastic scattering cross sections (in cm^2)	0 - 3500	20
39.	PPDC	6426	cumulative sampling distribution in $\cos\theta$ for π^+ - proton differential cross sections	0 - 2500	20
40.	PMDD	6426	cumulative sampling distribution in $\cos\theta$ for π^- - proton differential scattering cross sections	0 - 2500	20
41.	PMDC	6426	cumulative sampling distribution in $\cos\theta$ for π^- - proton differential charge exchange cross sections	0 - 2500	20
42.	PMDD	6426	cumulative sampling distribution in $\cos\theta$ for π^+ - proton differential scattering cross sections	0 - 2500	20

C.2.1 The CRSC Array

The first array, CRSC, contains 10 values for each target Mass number, A , from $A=1$ through 240. (Values for $A=240$ are not meaningful, although values are stored just to complete the record). For each A , the following values are stored (where $i=1,2,3$):

Word

Designation

Description

- | | |
|--------|--|
| 1 | A , target mass number |
| 2 - 4 | R_i , the radial boundaries of nucleus A , dividing the nucleus into 3 regions - a central sphere and two surrounding annuli (in cm) |
| 5 - 7 | ρ_i/A , nucleon densities per nucleon for each region, divided by an arbitrary normalization factor of 10^{30} (in cm^{-3}) |
| 8 - 10 | E_i^f , Fermi energy per nucleon for each region (in MeV) |

The radial boundaries are defined as those radii where the nucleon density has decreased to a specified value of its central value. A continuous radial dependence of nucleon densities are assumed /2/:

$$\rho(r) = \rho_1 \cdot [\exp((r-c)/Z_1) + 1]^{-1}$$

where ρ_1 =normalization factor, $c=1.07 \times 10^{-13} \cdot A^{1/3}$ cm, and $Z_1=0.545 \times 10^{-13}$ cm. The R_i are then defined by $\rho(R_i)=\alpha_i \cdot \rho(0)$, with $\alpha_1=0.90$, $\alpha_2=0.20$, and $\alpha_3=0.01$.

The number of neutrons or protons in each region is assumed to be the same as for the nucleus as a whole, that is,

$$\begin{aligned} n_i(N) &= \text{neutron density (per } 10^{30}) \text{ in region } i \\ &= (\rho_i/A) \cdot N, \text{ and} \end{aligned}$$

$$\begin{aligned} n_i(Z) &= \text{proton density (per } 10^{30}) \text{ in region } i \\ &= (\rho_i/A) \cdot Z \end{aligned}$$

In the Fermi gas model, where the nucleus is considered as a collection of free nucleons, the Fermi momentum of particles bound in volume Ω is conventionally written as (e.g., /3/),

$$P_f = (3\pi^2)^{1/3} \cdot h/2\pi \cdot (A/\Omega)^{1/2}$$

and the corresponding kinetic energy is $E^f = (P_f)^2/2M$. The input nucleon densities correspond to $\rho_i/A=1/\Omega$, and the input Fermi energies (per nucleon) correspond to

$$\hat{E}_i^f = E^f/A = (3h^3\rho_i/8\pi)/2M$$

The Fermi energies of neutrons and protons in each region then are

$$E_i^f(N) = \hat{E}_i^f \cdot (A-Z)^{2/3} \quad \text{and} \quad E_i^f(Z) = \hat{E}_i^f \cdot Z^{2/3}$$

C.2.2 Differential Cross-Section Data

The differential cross section data are represented in one of three formats: (a) as expressions of the form $A+B(\cos\theta)^n$ where the coefficients are input data, (b) as tabulated values corresponding to the areas under the angular distribution in $\Delta\cos\theta$ intervals, or (c) as tabulated cumulative frequency distributions in $\mu=\cos\theta$. The method used for each cross section is given in Table C.3.

Table 3. Methods used in representing differential scattering cross sections.

Cross Sections	Energy (MeV)	Method	Array
$\frac{d\sigma}{d\Omega}(n-p)$	0-300	(a) forward scattering: $A_F + B_F (\cos\theta)^2$ (b) backward scattering: $A_B + B_B (\cos\theta)^2$	DCLN
	300-740	(a) forward scattering: $A_F + B_F (\cos\theta)^2$ (b) backward scattering: $A_B + B_B (\cos\theta)^2$	DCIN
	660-3520	tabulated values for $\Delta\mu$ intervals, $\mu \geq 0$	DCHNA
	660-3520	tabulated values for $\Delta\mu$ intervals, $\mu \leq 0$	DCHNB
$\frac{d\sigma}{d\Omega}(p-p)$	0-500	(assumed isotropic in center-of-mass)	---
	440-1000	tabulated values for $\Delta\mu$ intervals	PDCI
	1000-4400	(as above)	PDCH
$\frac{d\sigma}{d\Omega}(\pi^+-p)$	0-2500	cumulative frequency distributions in μ , determined from phase shift analysis	PPDC
$\frac{d\sigma}{d\Omega}(\pi^0-p)$	0-2500	(as above)	PNDD
$\frac{d\sigma}{d\Omega}(\pi^0-p)$	0-2500	(as above)	PNDD
$\frac{d\sigma}{d\Omega}(\pi^0-p \text{ c.e.})$	0-2500	(as above)	PMDX

For n-p scattering, forward and backward scattering are treated separately, with the per cent of scattering in the backward direction specified by the DMHN data array. For p-p scattering, the angular distribution is assumed symmetric about $\mu=0$.

The pion-proton scattering and charge exchange cross sections are based on the phase shift data of /4/ and the formalism given by Bertini /5/. At each energy, values of μ defined by

$$R = \int_{-1}^{\mu} p(\mu) d\mu$$

are tabulated corresponding to $R=0$ to 1 in steps of 0.02.

C.2.3 Data for the Isobar Model

For single pion production collisions produced by an incident particle having relative kinetic energy E_r , the probability of creating an excited isobar of mass m is, according to the Sternheimer-Lindenbaum model /6/

$$P(m, E_r) = C \cdot \sigma_{3/2}(m) \cdot F(m, E_r)$$

where C is a normalization constant, $\sigma_{3/2}(m)$ is the total cross section for the pion-nucleon system (evaluated at the pion kinetic energy corresponding to the total system center-of-mass energy m), and F is the phase space available. The arrays FMXSN and FMXSP pertain to selecting from the F distribution for incident nucleons and pions, respectively. The arrays FRINN and DMIN are used in selecting from σ for nucleon-nucleon collisions, and FRIPN and PNMI are used in selecting from σ for pion-nuclear collisions.

For double pion production, the masses of the isobars are expressed as

$$P(m_1, m_2, E_r) = k \cdot \sigma_{3/2}(m_1) \cdot \sigma_{3/2}(m_2) \cdot F(m_1, m_2, E_r)$$

and the FMXDN array contains data needed in selecting from F .

The three remaining input arrays containing data used by the isobar pion production model are FSLN, PCFSL, and PNFSL, which are related to specifying final states. FSLN is the probability of two isobars in nucleon-nucleon collisions, PCFSL is for π^- -proton final states, and PNFSL is for π^0 -proton final states.

C.3 Plots of Input Data

Some of the input data are shown graphically in Figs. C.1 to C.8. The points shown in these graphs are the data points of the input data -- e.g., the data points vs. energy are at the energy values of the input data without any extrapolation or interpolation.

Figs. C.1 to C.3 show the contents of the CRSC input array. The total cross sections for all reactions are shown in Figs. C.4 to C.8.

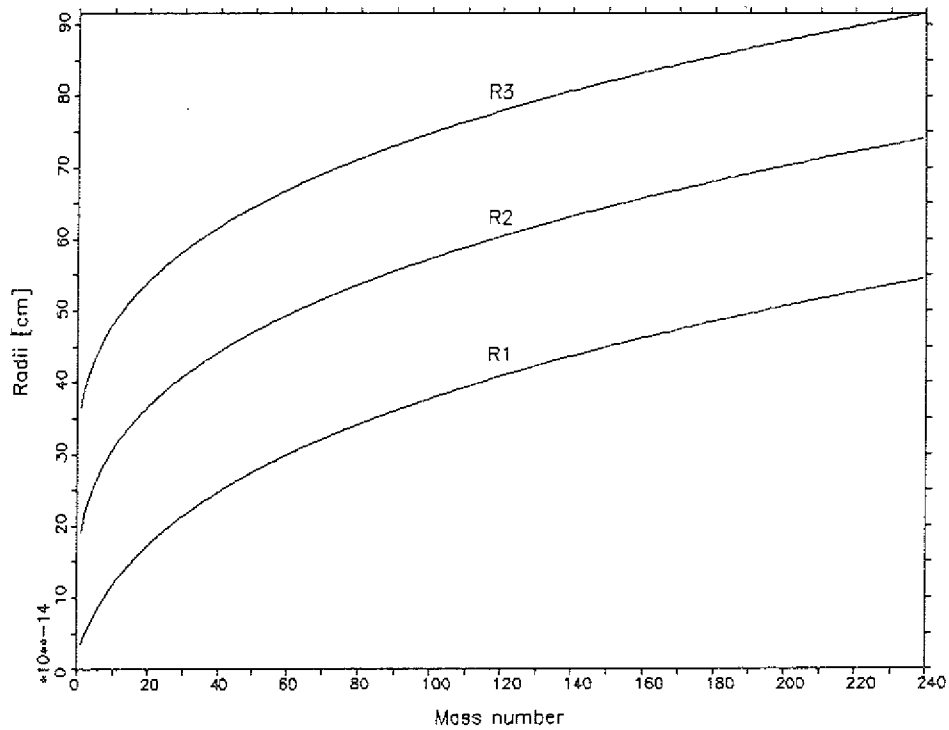


Fig. C.1: Input data (CRSC array) for radial boundaries defining regions of the nucleus. (Input values have been multiplied by 10^{13} before plotting)

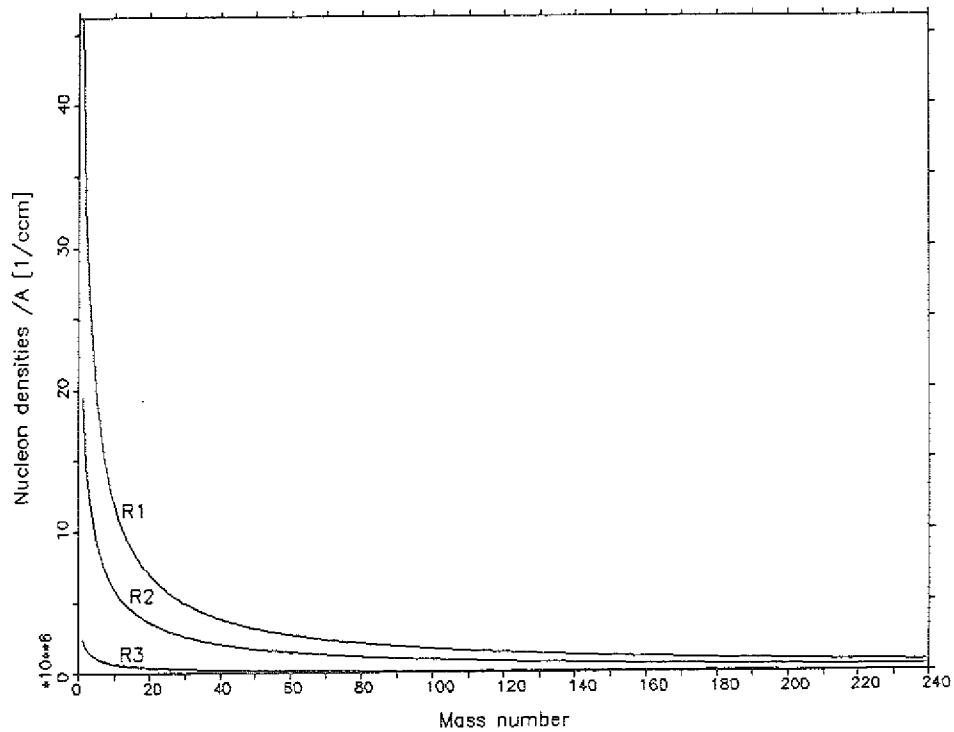


Fig. C.2: Input data (CRSC array) for nucleon density in each region of the nucleus. (The input values, and the values plotted, are divided arbitrarily by 10^{30})

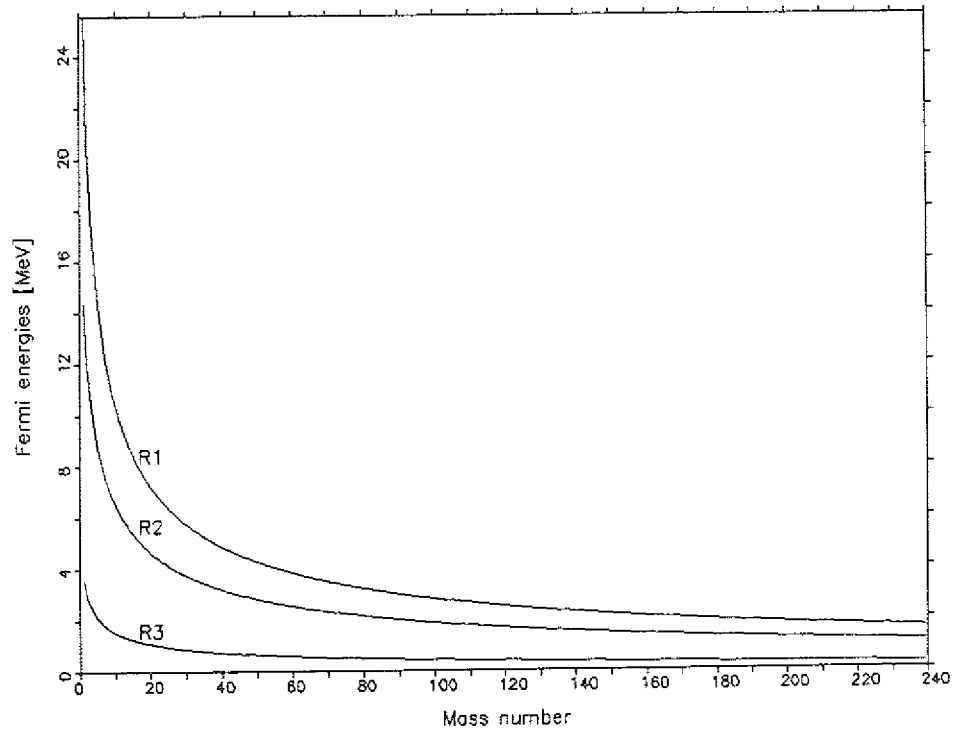


Fig. C.3: Input data (CRSC array) for Fermi energy per nucleon in each region of the nucleus

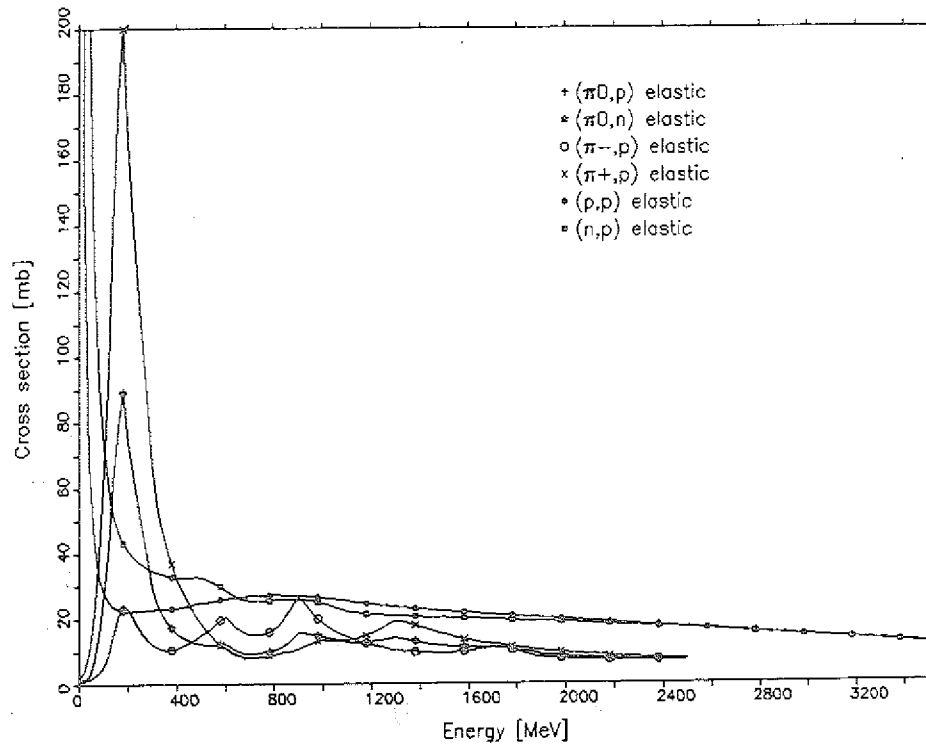


Fig. C.4: Input data used for pion-nucleon and nucleon-nucleon elastic scattering cross sections. (The low-energy region is shown expanded in Fig. C.5)

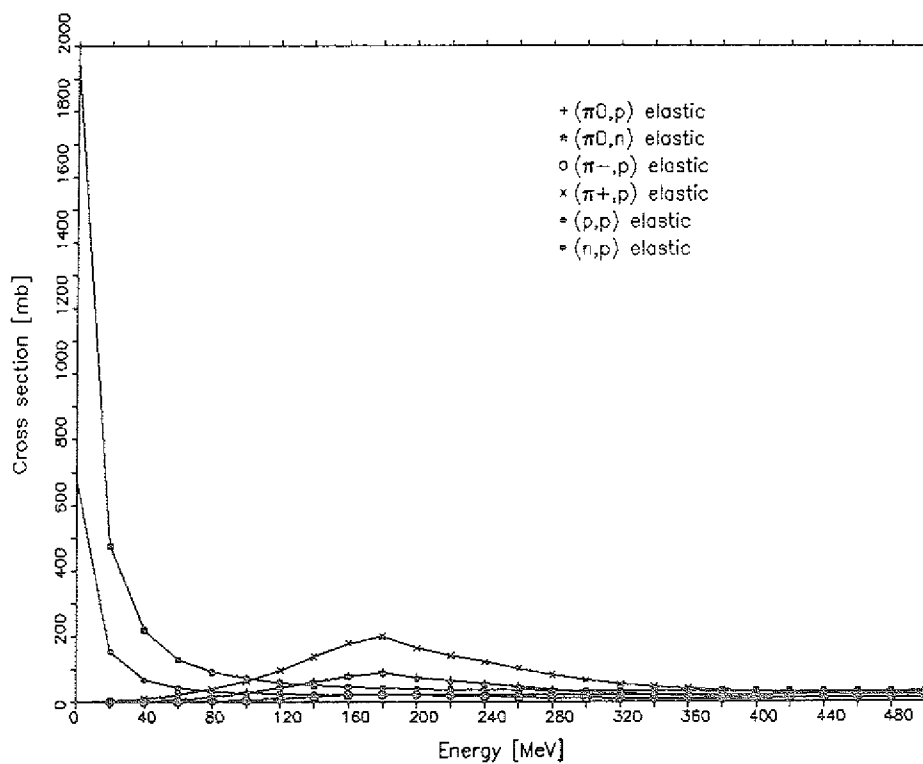


Fig. C.5: Pion-nucleon and nucleon-nucleon elastic scattering cross sections in the 0 to 500 MeV energy region

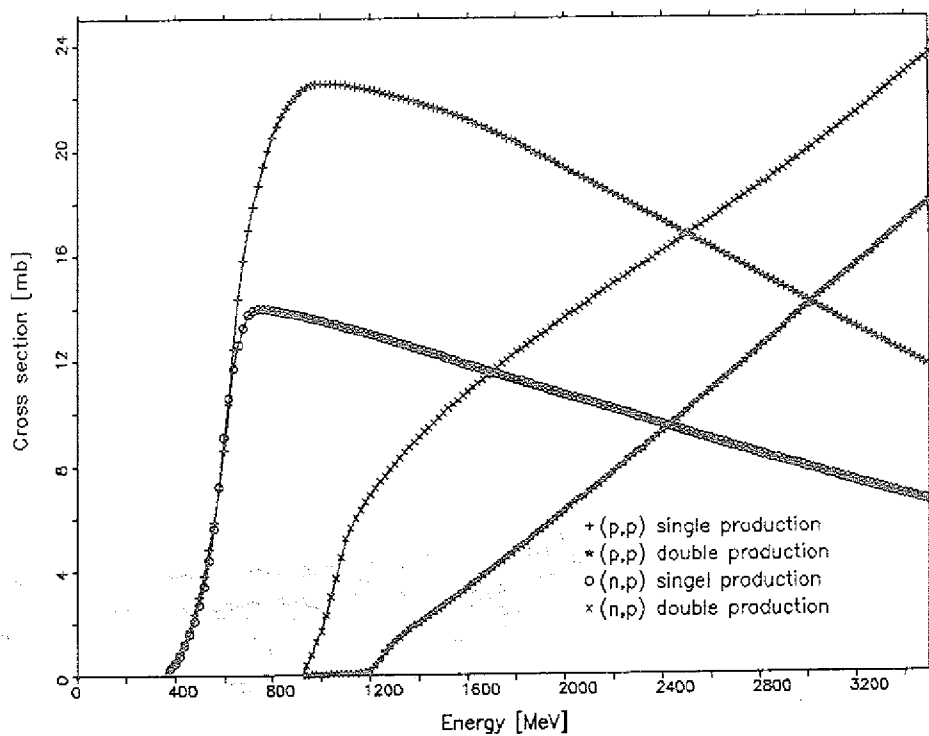


Fig. C.6: Input data used for pion production cross sections for nucleon-nucleon collisions

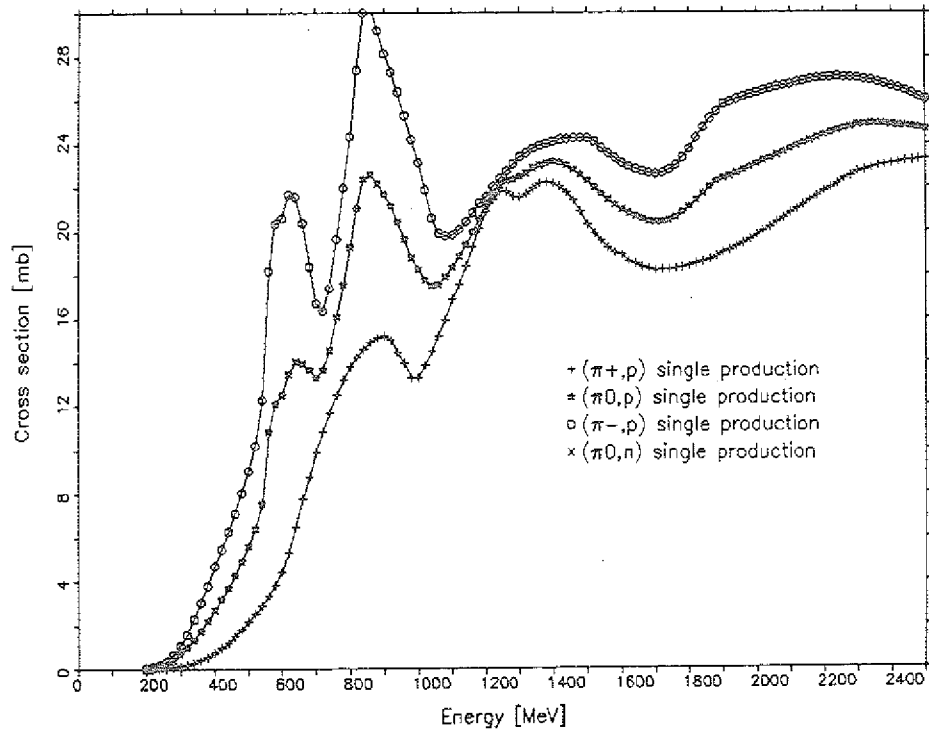


Fig. C.7: Input data used for pion production cross sections for pion-nucleon collisions

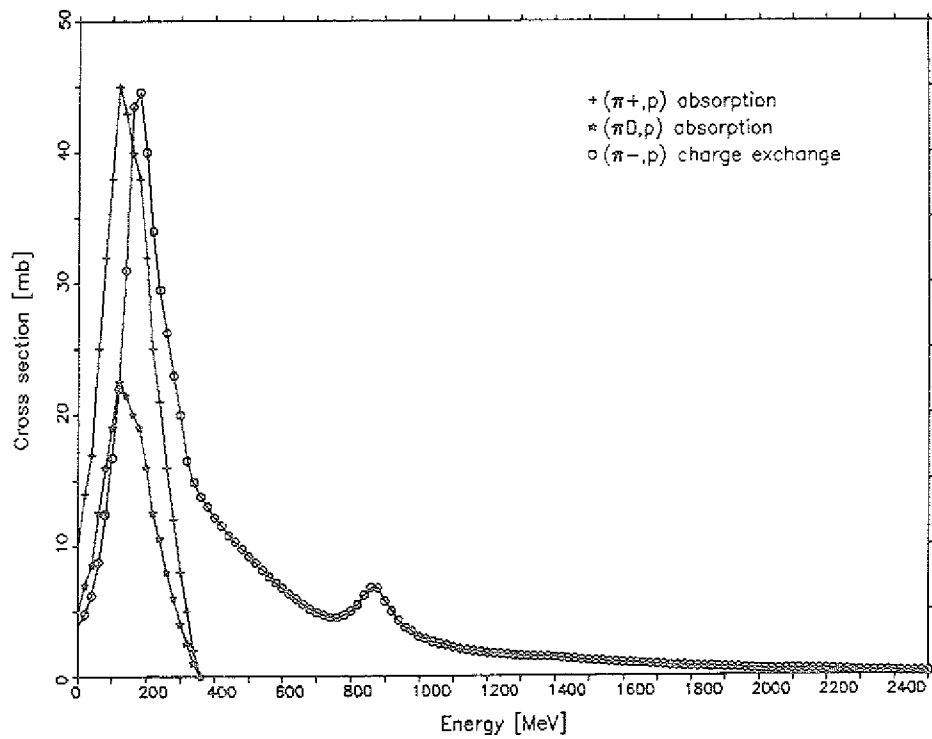


Fig. C.8: Input data used for pion absorption and charge-exchange cross sections

C.4 References Appendix C

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APPENDIX D
USER INTERFACING

D.1 Graphic Software Interface

The CMD command interpreter used by the HERMES system includes an interface to the KFA graphic software called the GR Software. The GR software is used to produce a series of viewgraphs. As this software is not portable it is necessary for the user, if he wants to use the graphic tools of HERMES, to provide a software which can be invoked by the following subroutine calls and performs the described functions.

The first call must be

CALL GRSTRT(35,8)

now unit 8 is used for plot output until

CALL GREND

terminates the graphic software. Whenever a viewgraph has been generated

CALL GRNXTF

displays the viewgraph on the terminal and/or plotter. The graphic memory is cleared and the GR Software is ready for the next viewgraph.

Scaling:

The dimension of a viewgraph is assumed to be 28.7*39.5 cm.

CALL GRSCLC(XLL,YLL,XUR,YUR)

defines a window with lower left corner at XLL, YLL and upper right corner XUR, YUR all measured in cm.

CALL GRSCLV(XA,YA,XB,YB)

defines the matching user values for the lower left and the upper right corners of the cm frame. The cm units refer to a plotter device with scaling factor 1, other devices need a scaling factor $\neq 1$ to fit 28.7*39.5 to the possible device coordinates. This may be included in the device driver software or can be set by calling

CALL GRFCTR(FACTOR)

Routines for simple drawing primitives are:

CALL GRJMP(X,Y)

move the pen to point X,Y (of the user value frame) without drawing

CALL GRDRW(X,Y)

draw a line from current position to X,Y

CALL GRJMPS(X,Y,ISY)

move the pen to X,Y without drawing and then draw symbol number ISY

CALL GRDRWS(X,Y,ISY)

draw a line from current pen position to X,Y and then draw symbol number ISY.

Polylines can be plotted with

CALL GRLN(X,Y,N)

move the cursor to X(1),Y(1) and draw lines to X(i),Y(i) for i=2 to N.

CALL GRCHN(X,Y,N,ISY)

draw polyline with symbol number ISY at each X(i), Y(i).
Text may be included with

CALL GRTXT(X,Y,LEN,Text)

at position X,Y. Additional routines are available to define drawing and text attributes:

CALL GRNWPEN(IPEN)

selects a new pen. This can be used to define colours e.g.

With

CALL GRFONT(ifont)

a character font can be selected and with

CALL GRCHRC(height,angle,intens)

the height of characters (in cm), the text angle and intensity can be selected. Finally,

```
GSTXAL(IA,4)
```

selects the text alignment as

```
IA=1  right justified
IA=2  centered
IA=3  left justified.
```

The subroutines described above form part of the low level graphics offered in the GR Software. Some additional routines of the second level are used by CMD as well. This second level provides automatic linear and logarithmic scaling and, therefore, needs FORTRAN unit number 4 to store data until all user data is available and automatic scaling can be done. We use the routines KURVEF and GRBLD.

```
CALL KURVEF(X,Y,N,ISTYLE)
```

defines a polyline or polymarker to be drawn and stores it on unit 4. X(N) and Y(N) are real arrays containing the coordinate data. ISTYLE defines how to draw the curves.

```
ISTYLE=(2*IPEN-1)*100+ISY
```

where ISY is the symbol number 1 to 13 or -1 for no symbol. IPEN denotes the colour of the curve. Thus, KURVEF can be used for polylines and polymarkers.

```
CALL GRBLD(XCM,YCM,IXSC,IYSC,XMIN,XMAX,YMIN,YMAX,KEY)
```

reads all data stored by KURVEF and draws all curves in one (or more) frames.

The parameters of GRBLD decide the appearance of the picture,

XCM,YCM are the lengths of x and y axis in cm coordinates.

On the first call GRBLD issues a

```
CALL GRSCLC(3.,35,3+XCM,3.5+YCM) and
```

```
CALL GRSCLV(0.,0.,XCM,YCM).
```

If XCM,YCM are given negative the lower left corner of the cm frame remains as it has been set by the user before calling GRBLD, and the upper right corner is set to (lower+left)+(XCM,YCM).

IXSC set the axis scaling
 IYSC 2 linear scale
 102 linear scale with grid lines
 -2 logarithmic scale
 -102 logarithmic scale with grid lines
 0 no axis (data is linear)

XMIN value frame used for user data
 XMAX 666. - use automatic minimum or maximum
 YMIN
 YMAX

KEY number of curves plotted in one frame.
 666 all curves, GRNXTF is called automatically
 -666 all curves, GRNXTF is not called
 use -666 to combine GRBLD with text drawing etc.

Some routines included in CMD extend the graphic interface providing some useful techniques of calling graphic for special purposes.

GRHEAD(TEXT)
 writes a chart title

GRXLAB(TEXT)
 writes a title at the x axis

GRYLAB(TEXT)
 writes a title at the y axis

GRLAB3(HEAD,XLAB,YLAB)
 writes the chart title and titles for the x and y axis

GRLGND(IPEN,ISY,ILIN,LTXT,TXT)
 add a line of text (of length LTXT) to the legend field of the chart using pen (colour) IPEN and prefixes the text optionally by a piece of line (if ILIN=1) and a symbol ISY

GRCURV(X,Y,N,IPEN,ISY,ILIN,LTXT,TXT)
 combines a call of KURVEF(X,Y,N,ISTYLE) with a call of GRLGND(IPEN,ISY,ILIN,LTXT,TXT)

GRHIST(X,Y,NY,ISTYLE)
 constructs a curve from X(0:N) and Y(N) data group entries and invokes KURVEF

GRBARS(X,Y,NY,IPEN,ISY,ILIN,LTXT,TXT)

combines a call to GRHIST with a call to GRLGND

GRFCMD

finally compiles the graphic commands and calls
the appropriate graphic routines.

D.2 Interfacing of User Provided Monte-Carlo Evaluation Codes

Interfacing for user written particle sources and analysis programs is possible, however, it has to be done in FORTRAN level.

The STATLIB subroutine package may provide programming aid for design of user detectors. For further details contact the authors.

APPENDIX E

SAMPLE PROBLEMS

Three examples of different HERMES use are given in this section. The necessary input is completely given following the guide for HERMES operation of Section 3.1. The sample outputs are reported as excerpts, because of their enormous length. They are given in full size on the HERMES distribution file.

E.1 Sample Problem I

High Energy Calorimeter

This is a simplified geometrical setup of a test calorimeter experiment, which is shown in detail in Fig. E.1 in a 9 tower multilayer structure. The simplified CG version is shown in Fig. E.2, where the 8 outer towers are considered as one, surrounding the central tower. The layer structure is also shown in this figure. A π^- beam of 5 GeV was assumed to hit the central tower in z-direction.

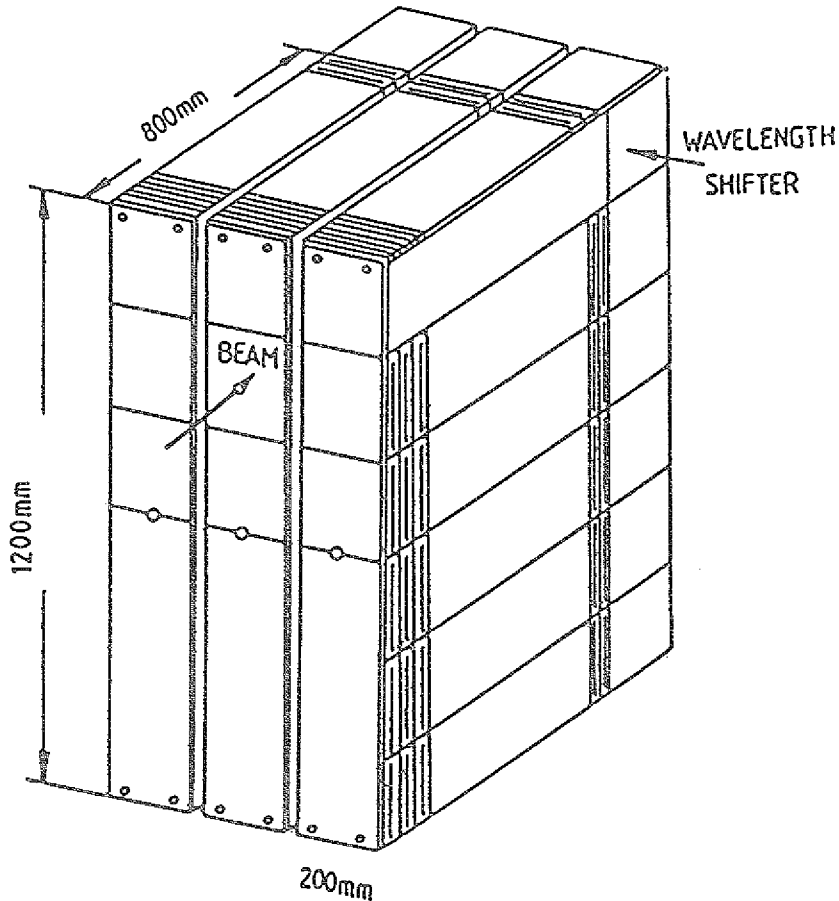


Fig. E.1: Experimental Setup

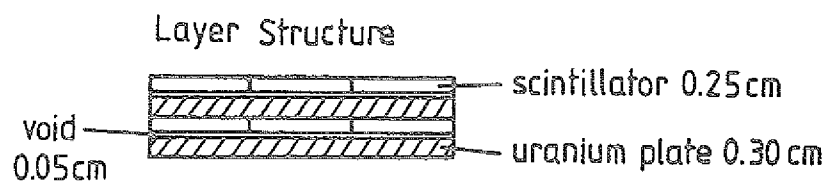
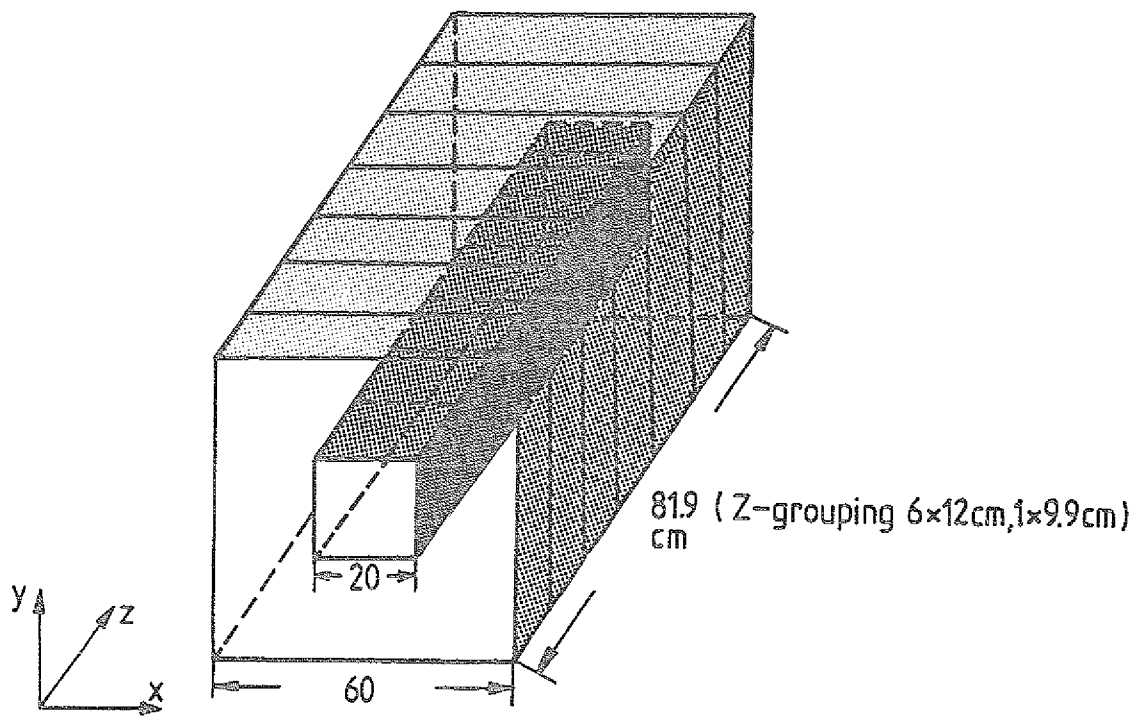


Fig. E.2: Geometrical Setup

E.1.1 Input Files for Sample Problem I

(1) Geometry Description in CMD Language

```

GEOCMP 412 435 435 4600 16000;
/ Define z structure body data
/ IBO body reference numbers
INTEGER IBO(410);
DATA IBO=<1000 A 59 1 2000 A 59 1 3000 A 59 1
        4000 A 59 1 5000 A 59 1 6000 A 59 1
        7000 A 48 1 8000> ;
/ ZB thickness of 409 z layers
REAL ZB(410);DATA ZB=<0. 0.3 0.05 0.25 Q 135 3 0.3 >;
/ Accumulate thicknesses to get coordinates
ZB(2:410)=ZB(1:409)+ZB(2:410);
/ Start body input with z structure
FOR I=1,410;
    PZ IBO(I) ZB(I);
ENDFOR;
/ Add two bodies to define the x-y-structure
RPP 500 -10. 10. -10. 10. -10. 90.;
RPP 501 -30. 30. -30. 30. -10. 90.;
END;
/ Start ZONE INPUT with course structure
ZONE -2 -1 500 +1000 -8000 GO 2; / Central tower
ZONE -2 -2 501 -500 +1000 -8000 GO 2; / Outer towers
ZONE -1 3 -501 +1000 -8000; / Side void
ZONE -1 4 -1000 ; / Front void
ZONE -1 5 +8000 ; / Rear void
/ Next define subgeometry data
/ low bounds of z-slabs are defined in IBO(1:409)
/ high bounds are defined by exclusion of bodies IBO(2:410)
INTEGER IB1(409); IB1=-IBO(2:410); / copy exclusion to IB1
/ define slab medium numbers
INTEGER IMO(409);DATA IMO=<1 6666 2 Q 135 3 1>;
/ define slab region number
/ all U will be region 1
/ all intermedium voids will be region 2
/ scintillator regions will be numbered as
/ Itower * 1000000 + Iz * 1000
INTEGER IRO(409);
DATA IRO=< 1 2 -1 Q 19 3
        1 2 -2 Q 19 3
        1 2 -3 Q 19 3
        1 2 -4 Q 19 3
        1 2 -5 Q 19 3
        1 2 -6 Q 19 3
        1 2 -7 Q 15 3
        1 >;
/ now setup subgeometry zones as defined above
SUBGEO;
FOR I=1,409;
    ZONE IMO(I) IRO(I) IBO(I) IB1(I);
ENDFOR;
/ End of geometry compilation
ENDGEO;
/ Load geometry object file from unit 22;
GEOIN;
/ Return to terminal input for test
UNIT 5;

```

(2) Input to HETC and SIM

```

//REW031H1 JOB ($$ACCOUNT$), 'TEST1-HET', TIME=(64), REGION=3000K,
// CLASS=C, MSGCLASS=A
/*JOBPARM L=15
/*PASSWORD $MVS
/*ROUTE PRINT CMS
//HET EXEC GFORTV, NAME=HETKFA2, DS='REW031.HERMES'
//G.FT20F001 DD DSN=REW071.HET86.BERT, DISP=SHR, LABEL=(,,IN) BERTP
//G.FT21F001 DD DSN=REW031.HETSUB.TEST1, DISP=SHR IOSUB
//G.FT22F001 DD DSN=REW031.HETDET.TEST1, DISP=SHR IOSIM
//G.FT96F001 DD DUMMY IODUM
//G.FT25F001 DD * INGEO
$INCLUDE TEST1 GEO-0
/*
//G.SYSIN DD *
  / request detector output for each batch to unit 22
SIMIO IOSIM 22;
  / load geometry object file from unit 25, print log. on unit 6
GEOIN 25,6;
  / define detectors
  / 1. scintillation light in any SCSN38 region
DEFDET EVIS,5,,,,,< 1001000 A 6 1000 2001000 A 6 1000 >;
  / 2. energy deposition all regions
DEFDET EDEP,4;
  / 3. energy leakage into side, front and rear void
DEFDET ELOS,9,,,,,<3 4 5>;
RUN; / return to het primary input
&RUN
  MAXCAS= 1, MAXBCH=1000,
  EMIN(1)=1., EMIN(2)=14.9, ELOP=20., EPICUT=2.23, EMUCUT=1.69,
  IBERTP=-20, IOSUB=21, NHSTP= 0, INGEO= 0, IOGEO= 6,
  IELAS=2, ISTRAG=0, IANG=0, IFISS=1, KEYDK=1,

  MXMAT=2,

  NEL(1)=2, DENH(1)=0., KB(1)=0.,
  ZZ(1,1)=92., A(1,1)=238., DEN(1,1)= 4.7932E-2,
  ZZ(2,1)=92., A(2,1)=235., DEN(2,1)= 1.4607E-4,

  NEL(2)=1, DENH(2)=4.8295E-2, KB(2)=8.5E-3,
  ZZ(1,2)= 6., A(1,2)= 12., DEN(1,2)= 4.8295E-2,

  TIPSR=4., ESR= 5000., EMAX=5000.,
  XSR=0., YSR=0., ZSR=0.001, USR=0., VSR=0., WSR=1.,

&END
EXIT;
/*

```

```

//REWI01$$ JOB ($$ACCOUNT$),TEST1,TIME=(128),REGION=6000K,CLASS=C
/*PASSWORD $MVS
/*JOBPARM L=20
/*ROUTE PRINT CHS
// EXEC IGFORTV
//LOAD DD DSN=REW031.HERMES,DISP=SHR
//L.SYSIN DD *
INCLUDE LOAD(MORSE)
INCLUDE LOAD(CGL2)
ENTRY MORSCC
//G.FT50F001 DD DSN=REW031.HERMES,TEST1,DISP=SHR,LABEL=(,IN)
//G.FT20F001 DD DSN=REW031.EPRLIB,DISP=SHR,LABEL=(,IN)
//G.FT21F001 DD DSN=REW101.TEST1.MORDET.IBM,DISP=SHR
//G.SYSIN DD *
HERMES TEST1 14 DETEKTOREN CARD A
0 0 32 6
113 2500 450 1 100 21 100 121 0 0 58 3 0 /
1 1 0 0 1.0E+0 1.0E-4 1.0E+4 1.0E-7 .6293E+6 /
CARD C
CARD B
CARD D
CARD D2
CARD F
/
ENERGIES (EV)
0.149E+08 0.135E+08 0.122E+08 0.111E+08 0.100E+08 0.905E+07 0.819E+07 /
0.741E+07 0.670E+07 0.607E+07 0.549E+07 0.497E+07 0.449E+07 0.407E+07 /
0.368E+07 0.333E+07 0.301E+07 0.273E+07 0.247E+07 0.222E+07 0.202E+07 /
0.183E+07 0.165E+07 0.150E+07 0.135E+07 0.122E+07 0.111E+07 0.100E+07 /
0.907E+06 0.821E+06 0.743E+06 0.672E+06 0.608E+06 0.550E+06 0.498E+06 /
0.450E+06 0.408E+06 0.369E+06 0.334E+06 0.302E+06 0.273E+06 0.247E+06 /
0.224E+06 0.202E+06 0.183E+06 0.166E+06 0.150E+06 0.136E+06 0.123E+06 /
0.111E+06 0.865E+05 0.674E+05 0.525E+05 0.409E+05 0.318E+05 0.248E+05 /
0.193E+05 0.150E+05 0.117E+05 0.912E+04 0.710E+04 0.553E+04 0.431E+04 /
0.335E+04 0.261E+04 0.203E+04 0.158E+04 0.123E+04 0.961E+03 0.749E+03 /
0.583E+03 0.454E+03 0.354E+03 0.275E+03 0.214E+03 0.167E+03 0.130E+03 /
0.101E+03 0.789E+02 0.614E+02 0.479E+02 0.373E+02 0.290E+02 0.226E+02 /
0.176E+02 0.137E+02 0.107E+02 0.832E+01 0.648E+01 0.504E+01 0.393E+01 /
0.306E+01 0.238E+01 0.186E+01 0.144E+01 0.113E+01 0.876E+00 0.683E+00 /
0.532E+00 0.414E+00 0.310E+00 0.240E+00 0.180E+00 0.140E+00 0.100E+00 /
0.700E+07 0.650E+07 0.600E+07 0.550E+07 0.500E+07 0.450E+07 0.400E+06 /
0.350E+07 0.300E+07 0.250E+07 0.200E+07 0.150E+07 0.100E+07 0.400E+06 /
0.200E+06 0.100E+06
000035FA731A
1 1 0 0 3 121 0
0 0 0 0 0 1.0E+0 1.0E+0 1.0E+0 1.0E+0 0.0E+0 /
-1
0 1 0 0
F 1. ;
FISSION WEIGHTS FVLO(MXREG)
CARD L
CARD M
FISSION SPEC. FSE(NMTG,MEDIA)
CARD N
.4122E-04 .1133E-03 .2770E-03 .6091E-03 .1218E-02 .2233E-02 .3791E-02 /
.6003E-02 .8930E-02 .1256E-01 .1679E-01 .2145E-01 .2632E-01 .3113E-01 /
.3564E-01 .3961E-01 .4289E-01 .4536E-01 .4696E-01 .4771E-01 .4766E-01 /
.4689E-01 .4551E-01 .4363E-01 .4133E-01 .3886E-01 .3618E-01 .3342E-01 /
.3065E-01 .2793E-01 .2531E-01 .2282E-01 .2048E-01 .1830E-01 .1630E-01 /
.1446E-01 .1280E-01 .1129E-01 .9941E-02 .8732E-02 .7654E-02 .6698E-02 /
.5852E-02 .5105E-02 .4448E-02 .3871E-02 .3365E-02 .2923E-02 .2536E-02 /
.4959E-02 .3458E-02 .2403E-02 .1666E-02 .1153E-02 .7967E-03 .5498E-03 /

```



```

.3791E-03 R 64 0.0 F 0. ; E O CARD N
R 100 1. R 21 0. Q 2 121 ; GAMMA PROB.'S GWLO(NMTG,MXREG) CARD O
MATERIALIEN EPR WQ P5 VON FT20 CARD XA
100 100 21 21 121 124 4 3 4 5 6 3 0/ CARD XB
0 0 0 0 0 0 20 0 0 0 0 0 0 / CARD XC
64 A 5 1 127 A 5 1 / C H
262 A 5 1 253 A 5 1 ; U238 U235 CARD XD
1 3 4.7932E-2 U238 CARD XF
1 -4 1.4607E-4 + U235
2 2 4.8295E-2 H
2 -1 4.8295E-2 + C
3 -2 4.8295E-2 H

ANALYSIS INPUT CARD AA
14 100 121 0 0 1 0 100 1 1/ CARD BB
VL
F 0. ; CARD CCX
F 0. ; CARD CCY
F 0. ; CARD CCY
1001000 A 6 1000 2001000 A 6 1000;
F 3 ;

CARD DD
/ VISIBLE ENERGY IN SCSN38 ; 7X INNER, 7X OUTER, 7X OUTER TOWER EXTRA
R 100 1. /
F 0. ;

CARD GG
1 A 120 1 ; CARD HH

1 / NUMBR
14 1. 0.0085 1 4.8295E-2 / NDETEC S KB NEL DENH
/ /ID = REGION NUMBERS
1001000 A 6 1000 2001000 A 6 1000;
F 6. ; ZZ
F 12. ; A
F 4.8295E-2 ; DEN
//G.FT32F001 DD *
412 435 435 4600 16000 0 0
4 0
0 1.000E+00
PZ 1000 0.000
PZ 1001 0.300
PZ 1002 0.350
PZ 1003 0.600
PZ 1004 0.900
PZ 1005 0.950
PZ 1006 1.200
PZ 1007 1.500
PZ 1008 1.550
PZ 1009 1.800
PZ 1010 2.100
PZ 1011 2.150
PZ 1012 2.400
PZ 1013 2.700
PZ 1014 2.750
PZ 1015 3.000
PZ 1016 3.300
PZ 1017 3.350
PZ 1018 3.600
PZ 1019 3.900
PZ 1020 3.950
PZ 1021 4.200
PZ 1022 4.500
PZ 1023 4.550
PZ 1024 4.800

```

PZ 1025 5.100
PZ 1026 5.150
PZ 1027 5.400
PZ 1028 5.700
PZ 1029 5.750
PZ 1030 6.000
PZ 1031 6.300
PZ 1032 6.350
PZ 1033 6.600
PZ 1034 6.900
PZ 1035 6.950
PZ 1036 7.200
PZ 1037 7.500
PZ 1038 7.550
PZ 1039 7.800
PZ 1040 8.100
PZ 1041 8.150
PZ 1042 8.400
PZ 1043 8.700
PZ 1044 8.750
PZ 1045 9.000
PZ 1046 9.300
PZ 1047 9.350
PZ 1048 9.600
PZ 1049 9.900
PZ 1050 9.950
PZ 1051 10.200
PZ 1052 10.500
PZ 1053 10.550
PZ 1054 10.800
PZ 1055 11.100
PZ 1056 11.150
PZ 1057 11.400
PZ 1058 11.700
PZ 1059 11.750
PZ 2000 12.000
PZ 2001 12.300
PZ 2002 12.350
PZ 2003 12.600
PZ 2004 12.900
PZ 2005 12.950
PZ 2006 13.200
PZ 2007 13.500
PZ 2008 13.550
PZ 2009 13.800
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PZ 2011 14.150
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PZ 2013 14.700
PZ 2014 14.750
PZ 2015 15.000
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PZ 2017 15.350
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PZ 2019 15.900
PZ 2020 15.950
PZ 2021 16.200
PZ 2022 16.500
PZ 2023 16.550
PZ 2024 16.800
PZ 2025 17.100
PZ 2026 17.150
PZ 2027 17.400

PZ 2028 17.700
PZ 2029 17.750
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PZ 2032 18.350
PZ 2033 18.600
PZ 2034 18.900
PZ 2035 18.950
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PZ 2037 19.500
PZ 2038 19.550
PZ 2039 19.800
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PZ 2041 20.150
PZ 2042 20.400
PZ 2043 20.700
PZ 2044 20.750
PZ 2045 21.000
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PZ 2047 21.350
PZ 2048 21.600
PZ 2049 21.900
PZ 2050 21.950
PZ 2051 22.200
PZ 2052 22.500
PZ 2053 22.550
PZ 2054 22.799
PZ 2055 23.099
PZ 2056 23.149
PZ 2057 23.399
PZ 2058 23.699
PZ 2059 23.749
PZ 3000 23.999
PZ 3001 24.299
PZ 3002 24.349
PZ 3003 24.599
PZ 3004 24.899
PZ 3005 24.949
PZ 3006 25.199
PZ 3007 25.499
PZ 3008 25.549
PZ 3009 25.799
PZ 3010 26.099
PZ 3011 26.149
PZ 3012 26.399
PZ 3013 26.699
PZ 3014 26.749
PZ 3015 26.999
PZ 3016 27.299
PZ 3017 27.349
PZ 3018 27.599
PZ 3019 27.899
PZ 3020 27.949
PZ 3021 28.199
PZ 3022 28.499
PZ 3023 28.549
PZ 3024 28.799
PZ 3025 29.099
PZ 3026 29.149
PZ 3027 29.399
PZ 3028 29.699
PZ 3029 29.749
PZ 3030 29.999

PZ 3031	30.299	PZ 4034	42.898
PZ 3032	30.349	PZ 4035	42.948
PZ 3033	30.599	PZ 4036	43.198
PZ 3034	30.899	PZ 4037	43.498
PZ 3035	30.949	PZ 4038	43.548
PZ 3036	31.199	PZ 4039	43.798
PZ 3037	31.499	PZ 4040	44.098
PZ 3038	31.549	PZ 4041	44.148
PZ 3039	31.799	PZ 4042	44.398
PZ 3040	32.099	PZ 4043	44.698
PZ 3041	32.149	PZ 4044	44.748
PZ 3042	32.399	PZ 4045	44.998
PZ 3043	32.699	PZ 4046	45.298
PZ 3044	32.749	PZ 4047	45.348
PZ 3045	32.999	PZ 4048	45.598
PZ 3046	33.299	PZ 4049	45.898
PZ 3047	33.349	PZ 4050	45.948
PZ 3048	33.599	PZ 4051	46.198
PZ 3049	33.899	PZ 4052	46.498
PZ 3050	33.949	PZ 4053	46.548
PZ 3051	34.199	PZ 4054	46.798
PZ 3052	34.499	PZ 4055	47.098
PZ 3053	34.549	PZ 4056	47.148
PZ 3054	34.799	PZ 4057	47.398
PZ 3055	35.099	PZ 4058	47.698
PZ 3056	35.149	PZ 4059	47.748
PZ 3057	35.399	PZ 5000	47.998
PZ 3058	35.699	PZ 5001	48.298
PZ 3059	35.749	PZ 5002	48.348
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PZ 4001	36.299	PZ 5004	48.898
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PZ 4003	36.599	PZ 5006	49.198
PZ 4004	36.899	PZ 5007	49.498
PZ 4005	36.949	PZ 5008	49.548
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PZ 4007	37.499	PZ 5010	50.098
PZ 4008	37.549	PZ 5011	50.148
PZ 4009	37.799	PZ 5012	50.398
PZ 4010	38.098	PZ 5013	50.698
PZ 4011	38.148	PZ 5014	50.748
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PZ 4013	38.698	PZ 5016	51.298
PZ 4014	38.748	PZ 5017	51.348
PZ 4015	38.998	PZ 5018	51.598
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PZ 4018	39.598	PZ 5021	52.198
PZ 4019	39.898	PZ 5022	52.498
PZ 4020	39.948	PZ 5023	52.548
PZ 4021	40.198	PZ 5024	52.798
PZ 4022	40.498	PZ 5025	53.098
PZ 4023	40.548	PZ 5026	53.147
PZ 4024	40.798	PZ 5027	53.397
PZ 4025	41.098	PZ 5028	53.697
PZ 4026	41.148	PZ 5029	53.747
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PZ 4028	41.698	PZ 5031	54.297
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PZ 4031	42.298	PZ 5034	54.897
PZ 4032	42.348	PZ 5035	54.947
PZ 4033	42.598	PZ 5036	55.197

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PZ 7002	72.346
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PZ 7017	75.346
PZ 7018	75.596
PZ 7019	75.896
PZ 7020	75.946
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PZ 7022	76.496
PZ 7023	76.546
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PZ 7026	77.146
PZ 7027	77.396
PZ 7028	77.696
PZ 7029	77.746
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PZ 7032	78.346
PZ 7033	78.596
PZ 7034	78.896
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PZ 7036	79.196
PZ 7037	79.496
PZ 7038	79.546
PZ 7039	79.796
PZ 7040	80.096
PZ 7041	80.146
PZ 7042	80.396

PZ	7043	80.696					
PZ	7044	80.746					
PZ	7045	80.996					
PZ	7046	81.296					
PZ	7047	81.346					
PZ	7048	81.596					
PZ	8000	81.896					
RPP	500	-10.000	10.000	-10.000	10.000	-10.000	90.000
RPP	501	-30.000	30.000	-30.000	30.000	-10.000	90.000
END							
-2	-1	500	1000	-8000GO	2		
-2	-2	501	-500	1000	-8000GO	2	
-1	3	-501	1000	-8000			
-1	4	-1000					
-1	5	8000					
777							
1	1	1000	-1001				
1000	2	1001	-1002				
2	-1	1002	-1003				
1	1	1003	-1004				
1000	2	1004	-1005				
2	-1	1005	-1006				
1	1	1006	-1007				
1000	2	1007	-1008				
2	-1	1008	-1009				
1	1	1009	-1010				
1000	2	1010	-1011				
2	-1	1011	-1012				
1	1	1012	-1013				
1000	2	1013	-1014				
2	-1	1014	-1015				
1	1	1015	-1016				
1000	2	1016	-1017				
2	-1	1017	-1018				
1	1	1018	-1019				
1000	2	1019	-1020				
2	-1	1020	-1021				
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1000	2	1022	-1023				
2	-1	1023	-1024				
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1000	2	1025	-1026				
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1000	2	1031	-1032				
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1000	2	1037	-1038				
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1000	2	1043	-1044				
2	-1	1044	-1045				
1	1	1045	-1046				
1000	2	1046	-1047				

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1000	2	1049	-1050
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1	1	1051	-1052
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1000	2	1058	-1059
2	-1	1059	-2000
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1000	2	2001	-2002
2	-2	2002	-2003
1	1	2003	-2004
1000	2	2004	-2005
2	-2	2005	-2006
1	1	2006	-2007
1000	2	2007	-2008
2	-2	2008	-2009
1	1	2009	-2010
1000	2	2010	-2011
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1000	2	2013	-2014
2	-2	2014	-2015
1	1	2015	-2016
1000	2	2016	-2017
2	-2	2017	-2018
1	1	2018	-2019
1000	2	2019	-2020
2	-2	2020	-2021
1	1	2021	-2022
1000	2	2022	-2023
2	-2	2023	-2024
1	1	2024	-2025
1000	2	2025	-2026
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1	1	2027	-2028
1000	2	2028	-2029
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1000	2	2031	-2032
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1000	2	2034	-2035
2	-2	2035	-2036
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1000	2	2037	-2038
2	-2	2038	-2039
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1000	2	2040	-2041
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2	-2	2044	-2045
1	1	2045	-2046
1000	2	2046	-2047
2	-2	2047	-2048
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1000	2	2049	-2050

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1000	2	2052	-2053
2	-2	2053	-2054
1	1	2054	-2055
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1000	2	2058	-2059
2	-2	2059	-3000
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1000	2	3001	-3002
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1000	2	3004	-3005
2	-3	3005	-3006
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1000	2	3007	-3008
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1000	2	3010	-3011
2	-3	3011	-3012
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1000	2	3013	-3014
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1000	2	3016	-3017
2	-3	3017	-3018
1	1	3018	-3019
1000	2	3019	-3020
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1000	2	3022	-3023
2	-3	3023	-3024
1	1	3024	-3025
1000	2	3025	-3026
2	-3	3026	-3027
1	1	3027	-3028
1000	2	3028	-3029
2	-3	3029	-3030
1	1	3030	-3031
1000	2	3031	-3032
2	-3	3032	-3033
1	1	3033	-3034
1000	2	3034	-3035
2	-3	3035	-3036
1	1	3036	-3037
1000	2	3037	-3038
2	-3	3038	-3039
1	1	3039	-3040
1000	2	3040	-3041
2	-3	3041	-3042
1	1	3042	-3043
1000	2	3043	-3044
2	-3	3044	-3045
1	1	3045	-3046
1000	2	3046	-3047
2	-3	3047	-3048
1	1	3048	-3049
1000	2	3049	-3050
2	-3	3050	-3051
1	1	3051	-3052
1000	2	3052	-3053

2	-3	3053	-3054
1	1	3054	-3055
1000	2	3055	-3056
2	-3	3056	-3057
1	1	3057	-3058
1000	2	3058	-3059
2	-3	3059	-4000
1	1	4000	-4001
1000	2	4001	-4002
2	-4	4002	-4003
1	1	4003	-4004
1000	2	4004	-4005
2	-4	4005	-4006
1	1	4006	-4007
1000	2	4007	-4008
2	-4	4008	-4009
1	1	4009	-4010
1000	2	4010	-4011
2	-4	4011	-4012
1	1	4012	-4013
1000	2	4013	-4014
2	-4	4014	-4015
1	1	4015	-4016
1000	2	4016	-4017
2	-4	4017	-4018
1	1	4018	-4019
1000	2	4019	-4020
2	-4	4020	-4021
1	1	4021	-4022
1000	2	4022	-4023
2	-4	4023	-4024
1	1	4024	-4025
1000	2	4025	-4026
2	-4	4026	-4027
1	1	4027	-4028
1000	2	4028	-4029
2	-4	4029	-4030
1	1	4030	-4031
1000	2	4031	-4032
2	-4	4032	-4033
1	1	4033	-4034
1000	2	4034	-4035
2	-4	4035	-4036
1	1	4036	-4037
1000	2	4037	-4038
2	-4	4038	-4039
1	1	4039	-4040
1000	2	4040	-4041
2	-4	4041	-4042
1	1	4042	-4043
1000	2	4043	-4044
2	-4	4044	-4045
1	1	4045	-4046
1000	2	4046	-4047
2	-4	4047	-4048
1	1	4048	-4049
1000	2	4049	-4050
2	-4	4050	-4051
1	1	4051	-4052
1000	2	4052	-4053
2	-4	4053	-4054
1	1	4054	-4055
1000	2	4055	-4056

2	-4	4056	-4057
1	1	4057	-4058
1000	2	4058	-4059
2	-4	4059	-5000
1	1	5000	-5001
1000	2	5001	-5002
2	-5	5002	-5003
1	1	5003	-5004
1000	2	5004	-5005
2	-5	5005	-5006
1	1	5006	-5007
1000	2	5007	-5008
2	-5	5008	-5009
1	1	5009	-5010
1000	2	5010	-5011
2	-5	5011	-5012
1	1	5012	-5013
1000	2	5013	-5014
2	-5	5014	-5015
1	1	5015	-5016
1000	2	5016	-5017
2	-5	5017	-5018
1	1	5018	-5019
1000	2	5019	-5020
2	-5	5020	-5021
1	1	5021	-5022
1000	2	5022	-5023
2	-5	5023	-5024
1	1	5024	-5025
1000	2	5025	-5026
2	-5	5026	-5027
1	1	5027	-5028
1000	2	5028	-5029
2	-5	5029	-5030
1	1	5030	-5031
1000	2	5031	-5032
2	-5	5032	-5033
1	1	5033	-5034
1000	2	5034	-5035
2	-5	5035	-5036
1	1	5036	-5037
1000	2	5037	-5038
2	-5	5038	-5039
1	1	5039	-5040
1000	2	5040	-5041
2	-5	5041	-5042
1	1	5042	-5043
1000	2	5043	-5044
2	-5	5044	-5045
1	1	5045	-5046
1000	2	5046	-5047
2	-5	5047	-5048
1	1	5048	-5049
1000	2	5049	-5050
2	-5	5050	-5051
1	1	5051	-5052
1000	2	5052	-5053
2	-5	5053	-5054
1	1	5054	-5055
1000	2	5055	-5056
2	-5	5056	-5057
1	1	5057	-5058
1000	2	5058	-5059

```

2 -5 5059 -6000
1 1 6000 -6001
1000 2 6001 -6002
2 -6 6002 -6003
1 1 6003 -6004
1000 2 6004 -6005
2 -6 6005 -6006
1 1 6006 -6007
1000 2 6007 -6008
2 -6 6008 -6009
1 1 6009 -6010
1000 2 6010 -6011
2 -6 6011 -6012
1 1 6012 -6013
1000 2 6013 -6014
2 -6 6014 -6015
1 1 6015 -6016
1000 2 6016 -6017
2 -6 6017 -6018
1 1 6018 -6019
1000 2 6019 -6020
2 -6 6020 -6021
1 1 6021 -6022
1000 2 6022 -6023
2 -6 6023 -6024
1 1 6024 -6025
1000 2 6025 -6026
2 -6 6026 -6027
1 1 6027 -6028
1000 2 6028 -6029
2 -6 6029 -6030
1 1 6030 -6031
1000 2 6031 -6032
2 -6 6032 -6033
1 1 6033 -6034
1000 2 6034 -6035
2 -6 6035 -6036
1 1 6036 -6037
1000 2 6037 -6038
2 -6 6038 -6039
1 1 6039 -6040
1000 2 6040 -6041
2 -6 6041 -6042
1 1 6042 -6043
1000 2 6043 -6044
2 -6 6044 -6045
1 1 6045 -6046
1000 2 6046 -6047
2 -6 6047 -6048
1 1 6048 -6049
1000 2 6049 -6050
2 -6 6050 -6051
1 1 6051 -6052
1000 2 6052 -6053
2 -6 6053 -6054
1 1 6054 -6055
1000 2 6055 -6056
2 -6 6056 -6057
1 1 6057 -6058
1000 2 6058 -6059
2 -6 6059 -7000
1 1 7000 -7001
1000 2 7001 -7002

```

```

2 -7 7002 -7003
1 1 7003 -7004
1000 2 7004 -7005
2 -7 7005 -7006
1 1 7006 -7007
1000 2 7007 -7008
2 -7 7008 -7009
1 1 7009 -7010
1000 2 7010 -7011
2 -7 7011 -7012
1 1 7012 -7013
1000 2 7013 -7014
2 -7 7014 -7015
1 1 7015 -7016
1000 2 7016 -7017
2 -7 7017 -7018
1 1 7018 -7019
1000 2 7019 -7020
2 -7 7020 -7021
1 1 7021 -7022
1000 2 7022 -7023
2 -7 7023 -7024
1 1 7024 -7025
1000 2 7025 -7026
2 -7 7026 -7027
1 1 7027 -7028
1000 2 7028 -7029
2 -7 7029 -7030
1 1 7030 -7031
1000 2 7031 -7032
2 -7 7032 -7033
1 1 7033 -7034
1000 2 7034 -7035
2 -7 7035 -7036
1 1 7036 -7037
1000 2 7037 -7038
2 -7 7038 -7039
1 1 7039 -7040
1000 2 7040 -7041
2 -7 7041 -7042
1 1 7042 -7043
1000 2 7043 -7044
2 -7 7044 -7045
1 1 7045 -7046
1000 2 7046 -7047
2 -7 7047 -7048
1 1 7048 -8000

```

```

999
/* E O F

```


(4) Input to NDEM (not applied in this problem)

(5) Input to PEGS

```
//REW031P1 JOB ($$ACCOUNT$),PEGS4,CLASS=C,MSGCLASS=A,REGION=2000K,
// TIME=(4)
/*JOBPARM L=15
/*PASSWORD $MVS
/*ROUTE PRINT CMS
// EXEC GFORTV,NAME=PEGS4,DS='REW031.HERMES'
//G.FT06F001 DD SYSOUT=*
//G.FT07F001 DD DSN=REW031.PEGS4.TEST1,DISP=SHR
//G.FT08F001 DD DSN=REW071.PEGS4.PEPR,DISP=SHR,LABEL=(,,,IN)
//G.FT09F001 DD DSN=REW071.PEGS4.FORM,DISP=SHR,LABEL=(,,,IN)
//G.FT10F001 DD SYSOUT=*,DCB=(RECFM=UA,LRECL=132)
//G.SYSIN DD * FT05F001
ENER
&INP AE=0.711,UE=5000.,AP=0.10,UP=5000., &END
COMP
&INP NE=2, RHO=1.044, PZ=1.,1.,
AFACT=0.,CBAR=0.,SK=0.,X0=0.,X1=0.,IEV=0.,
&END
SCSN38
H C
PWLF
&INP &END
DECK
&INP &END
MIXT
&INP NE=2, RHO=19., RHOZ= 99.97,0.03, WA=238.,235.,
AFACT=0.,CBAR=0.,SK=0.,X0=0.,X1=0.,IEV=0.,
&END
URAN
U U
PWLF
&INP &END
DECK
&INP &END
STOP
&INP &END
/*
```

(6) Input to EGS (for π^0 -processing)

```

//REW031$$ JOB ($$ACCOUNT$),'TEST1-EGS4',CLASS=C,MSGCLASS=A,
// REGION=2000K,TIME=(128)
/*JOBPARM L=15
/*PASSWORD $MVS
/*ROUTE PRINT CMS
/** -----
/** EGS
/** -----
// EXEC GFORTV,COND=(9,LT),NAME=EGS4,DS='REW031.HERMES'
//G.SYSIN DD *
&EGS
  IRUN=1, MHIS=1, MBAT= 500, IXX=123456789,
  INEGS=1,IOEGS=2, INGEO=5,IOGEO=6,
  NMED=2,
  MEDNAM= 'URAN', 'SCSN38',
  ESTEPE= 0.005, 0.01,
  NREG=19,
  LSTREG= 1001000,1002000,1003000,1004000,1005000,1006000,1007000,
          2001000,2002000,2003000,2004000,2005000,2006000,2007000,
          1, 2, 3, 4, 5,
  ECUT= 7*1.511, 7*1.511, 1.511, 4*0.,
  PCUT= 7*0.100, 7*0.100, 0.100, 4*0.,
  MED= 7*2, 7*2, 1, 4*0,
  IRAYLR= 19*0,
  IQI=-1, EI=5.E3, XI=0.,YI=0.,ZI=0.01, UI=0.,VI=0.,WI=1.,WTI=1.,
&END
$INCLUDE TEST1 GEO-O
/*
//G.FT01F001 DD DSN=REW031.HETSUB.TEST1,DISP=SHR SOURCE FILE
//G.FT02F001 DD DSN=REW031.EGSP10.TEST1,DISP=SHR EGS DETECTOR FILE
//G.FT12F001 DD DSN=REW031.PEGS4.TEST1,DISP=SHR HATCH INPUT
//G.FT08F001 DD DUMMY

```

(7) Input to STATIST

```

//REW031$$ JOB ($$ACCOUNT$), 'TEST1-STATIST', TIME=(,30), REGION=4000K,
// CLASS=C, MSGCLASS=A
/*JOBPARM L=15
/** test1 statist
/*PASSWORD $MVS
/*ROUTE PRINT CMS.REW031
// EXEC GFOR TV, DS='REW031.HERMES', NAME=STATPRT
/**      grafic workspace ( line printer grafic )
//G.FT04F001 DD UNIT=TEMP, SPACE=(TRK,(10,2)), DISP=(NEW,DELETE)
/**      grafic output      ( line printer grafic )
//G.FT08F001 DD SYSOUT=L, DEST=(CMS,REW031)
/**      submission files
//G.FT10F001 DD DSN=REW031.HETDET.TEST1, DISP=SHR
//G.FT11F001 DD DSN=REW031.MORDET.TEST1, DISP=SHR
//G.FT12F001 DD DSN=REW031.EGSPIO.TEST1, DISP=SHR
//G.SYSIN DD *
/ Test1 STATIST
/ 1) define data arrays for STATIST analysis
/ 1.1) setup arrays to read in HET, MORSE, and EGS(pi0) detector records
REAL EDEP(19);
REAL FHET(14,19), FMOR(14), FPIO(19,3,5);
/ 1.2) setup arrays to keep the sums
/ over all contribution types of interest
REAL EHET(14), EMOR(14), EPIO(19);
/ 1.3) setup weight arrays used to extract the contributions of interest
REAL WHET(19), WMOR(1), WPIO(15);
DATA WHET=< 1. 1. 1. F 0.>;
DATA WMOR=1.;
DATA WPIO=< R 3 1. R 3 1. R 3 1. R 3 0. 0. 1. 0.>;
/ 1.4) setup an array to sum over all scintillator regions
REAL SCINT(19); DATA SCINT=<R 14 1. F 0.>;
/ 1.5) setup variables for final event results
REAL EVCHG(1), EVPIO(1), EVNEU(1);
/ 2) start event processing (7 variables 450 events)
STATIN 7,450;
/ 2.1) load detector records from submission files
LOAD 10,EVIS@EOB,FHET; / HET DIFFERENTIAL PULSE DISTRIBUTION
LOAD 10,EDEP@EOB,EDEP; / HET GLOBAL ENERGY DEPOSITION
LOAD 11,R001@EOB,FMOR; / MORSE VISIBLE N ENERGY
LOAD 12,EDEP@EOB,FPIO; / EGS VISIBLE PIO ENERGY
/ 2.2) extract main contributions
DOT EHET=EHET*WHET; / EXTRACT EVIS
DOT EMOR=FMOR*WMOR; / EXTRACT EVIS
DOT EPIO=FPIO*WPIO (19,15,1); / EXTRACT EVIS
/ 2.3) sum over all scintillator regions
DOT EVCHG=EHET*SCINT (1,14,1);
DOT EVNEU=EMOR*SCINT (1,14,1);
DOT EVPIO=EPIO*SCINT (1,19,1);

/ 2.4) assign V containing 7 variables
/ for each event, V is stacked into EVENT
SET V( 1)=EDEP(1)+EDEP(2)+EDEP(3);
SET V( 2)=EDEP(17);
SET V( 3)=EDEP(18);
SET V( 4)=EVCHG(1);
SET V( 5)=EVNEU(1);
SET V( 6)=EVPIO(1);
SET V( 7)=V(4)+V(5)+V(6);
/ 3) start end of run analysis
BIN 1,0.,5000., 50, ' Edep p,pi+,pi-';
BIN 2,0.,1000., 50, ' Esub n';
BIN 3,0.,5000., 50, ' Esub pi0';

```

```
BIN 4,0.,500.0, 50,' Evis p,pi+,pi-';  
BIN 5,0.,100.0, 50,' Evis n';  
BIN 6,0.,500.0, 50,' Evis pi0';
```

```
BIN 7,0.,500.0, 50,' Evis p,pi+,pi-,n,pi0';  
/ NOTE: FIT and SCAT require a previous call of BIN  
/ for the variables wanted.
```

```
FIT 7;  
SCAT 1,4;  
SCAT 2,5;  
SCAT 3,6;  
STATIST PLOT;  
EXIT;
```

```
/*
```

E.1.2 Output Files Excerpt for Sample Problem I

(1) Geometry Object File on Tape

(2) Output of HETC and SIM

```

/JOB-ID = REW031H1 05/03/88 12:36:16
I: / request detector output for each batch to unit 22
I: SIMIO IOSIM 22;
I: / load geometry object file from unit 25, print log. on unit 6
I: GEOM 25,6;
  NUMB. OF BODIES (NUMB)           412
  NUMB. OF REGIONS (NUMR)          435
  NUMBER OF CODE REGIONS (NCOD)     435
  LENGTH OF FDP ARRAY (LMDX)        4600
  LENGTH OF MA ARRAY (LMDY)         16000
  MODIFIED INPUT (LOUT)              0
  VOLUME INPUT OPTION (IVOPT)        4 NOT USED
  BOUNDARY OPTION (MEDREG)           0
  DEBUG OPTION (IDBGO)               0

```

MACHINE EPS = 0.100000E-07

```

IPRINT= 0
STRETCH= 1.0000000E+00

```

```

NUMBER OF BODIES           412
LENGTH OF MA -ARRAY        2885
LENGTH OF FDP-ARRAY        4543
SUB-GEOMETRY #2
NUMBER OF INPUT ZONES       414
NUMBER OF CODE ZONES        414
LENGTH OF INTEGER ARRAY 6619
I: / define detectors
I: / 1. scintillation light in any SCSN38 region
I: DEFDET EVIS,5,,,,,< 1001000 A 6 1000 2001000 A 6 1000 >;
I: / 2. energy deposition all regions
I: DEFDET EDEP,4;
I: / 3. energy leakage into side, front and rear void
I: DEFDET ELOS,9,,,,,<3 4 5>;
I:RUN; / return to hot primary input
MAXBCH=1000 IBERTP=-20 EMIN(1)= 1.000E+00 ELOP = 2.000E+01
MAXCAS= 1 IOSUB = 21 EMIN(2)= 1.490E+01 CTOFE = 0.000E+00
IRUN = 1 NHSTP = 0 EPICUT = 2.230E+00 EMAX = 5.000E+03
          ENUCUT = 1.690E+00 BO = 8.000E+00
NICOL =0 NPIDK =1 NBOGUS =1 NSEUDO=1
NSPRD=0 NWSPRD=0 ISTRAC =0 IANG =0
IFISS =1 IBO =1 IGEOM =0 KEYDK =1
IELAS =2
ISEED = 123456789
LHI =0 HIA(5)= 0.0 HIZ(5)= 0.0 HIBE(5)= 0.000E+00
/BERTP(1): 1-N-C DATA READ FROM UNIT 20
/BERTP(1): EVAP DATA READ FROM UNIT 20
FISSION OPTION SELECTED
HSIGMX 0.12000E-23 0.64942E-24 0.20000E-24 0.00000E+00 0.67700E-25
THIS IS MEDIUM 1
DENH=0.000000E+00 NEL= 2 SO= 1.000E+00 KB= 0.000E+00
ZZ( 1, 1)=92.0 A( 1, 1)=238.0 DEN( 1, 1)=4.793200E-02
ZZ( 2, 1)=92.0 A( 2, 1)=235.0 DEN( 2, 1)=1.460700E-04
THIS IS MEDIUM 2
DENH=4.829500E-02 NEL= 1 SO= 1.000E+00 KB= 8.500E-03
ZZ( 1, 2)= 6.0 A( 1, 2)= 12.0 DEN( 1, 2)=4.829500E-02
"GEOMETRIC" X-SECTIONS IN (1/CM)

```

NUCLIDS:	1	2	3	4	5	6	7	8	9	10
MEDIA										
1	1.2588E-01	3.8128E-04								
2	3.7396E-02									

MAXIMUM M-1 X-SECTIONS IN (1/CM)

MEDIA	PROTONS	NEUTRONS	PI+	PI0	PI-
1	0.0000E+00	0.0000E+00	0.0000E+00	0.0000E+00	0.0000E+00
2	5.7954E-02	3.1364E-02	9.6590E-03	0.0000E+00	3.2696E-03

TRANSPORT X-SECTION = SUM("GEOMETRIC") + MAX. H-1 + MAX. DECAY X-SECTIONS IN (1/CM)

MEDIA	PROTONS	NEUTRONS	PI+	PI0	PI-	MUE+	MUE-
1	1.2626E-01	1.2626E-01	1.3329E-01	0.0000E+00	1.3329E-01	8.8943E-05	8.8943E-05
2	9.5350E-02	6.8759E-02	5.4078E-02	0.0000E+00	4.7689E-02	8.8943E-05	8.8943E-05

SOURCE DATA:

TYP = 4.
 XYZ = 0.000 0.000 0.001 GAUSSIAN DISTRIBUTED WITH
 XYZ SIG. = 0.000 0.000 0.000 TRUNCATED TO RADIUS
 R = 0.000 IF R>0
 UVW = 0.0000 0.0000 1.0000 IF = 0.,0.,0. -> ISOTROPIC
 WT = 1.000
 E = 5000.000 GAUSSIAN DISTRIBUTED WITH
 E SIG. = 0.000

M:GOMIN INCEO= 0 IOCEO= 6

```

--> SOR HET  IRUN= 1 ISEED= 123456789
/SUBMIT(1): IOSUB=21
/HET1(1): SKALING TURNED ON
--> SOB HET  IBAT= 1 ISEED= 123456789
/BERTTP(1): I-N-C DATA READ FORM UNIT 20
/DRESF(1): USING 1977 WAPS DATA
--> EOB HET  IBAT= 1 ISEED= 1486727229 NHIS= 1 CPU= 27.05SEC.
MEDIUM SORS CASC SLOW ESCP PSEU ABSO BRDY SCAT

```

```

1 1 61 9 3 10 6 717 0 0
2 0 17 2 2 11 1 728 0 0
6666 0 0 0 0 1 0 722 0 0
TOTAL 1 78 11 5 22 7 2167 0 0
/SUBMIT(1): NEUTRONS: 156, PIO: 2, NUCLEI: 24
-----

```

```

--> SOB HET  IBAT= 2 ISEED= 1486727229
--> EOB HET  IBAT= 2 ISEED= 1453333397 NHIS= 2 CPU= 2.71SEC.
/SUBMIT(1): NEUTRONS: 102, PIO: 2, NUCLEI: 22
-----

```

```

--> EOR HET  IRUN= 1 ISEED= 606246303 NHIS= 1000 CPU= 1877.56SEC.
MEDIUM SORS CASC SLOW ESCP PSEU ABSO BRDY SCAT

```

```

1 1000 29539 13443 3009 6137 3474 375957 0 0
2 0 6128 2665 696 5767 115 380467 0 0
6666 0 0 0 145 201 0 377238 0 0
TOTAL 1000 35667 16108 3850 12105 35891133662 0 0

```

		REAL NONHYDROG COLLISIONS	PSEUDO NONHYDROG COLLISIONS	REAL HYDROGEN COLLISIONS	PSEUDO HYDROGEN COLLISIONS	PSEUDO DECAY COLLISIONS	REAL ELASTIC COLLISIONS	PSEUDO ELASTIC COLLISIONS
MEDIUM	1	0.15077E+05	0.58720E+04	0.00000E+00	0.00000E+00	0.16600E+03	0.17936E+05	0.96000E+02
MEDIUM	2	0.20560E+04	0.29300E+04	0.13570E+04	0.27060E+04	0.11300E+03	0.28300E+04	0.17000E+02

PSEUDO COLLISIONS WITHIN INTERNAL VOID BY PARTICLES OF

	TYPE 1	TYPE 2	TYPE 3	TYPE 4	TYPE 5	TYPE 6	TYPE 7
	0.15000E+02	0.16200E+03	0.30000E+01	0.00000E+00	0.21000E+02	0.00000E+00	0.00000E+00
****	COLLISION INFORMATION FOR NUCLEONS ABOVE			3495.0	MEV AND CHG. PIONS ABOVE	2495.0	MEV.****
	REAL NONHYDROG COLLISIONS	PSEUDO NONHYDROG COLLISIONS	REAL HYDROGEN COLLISIONS	PSEUDO HYDROGEN COLLISIONS	PSEUDO DECAY COLLISIONS	REAL ELASTIC COLLISIONS	PSEUDO ELASTIC COLLISIONS
MEDIUM	1	0.10020E+04	0.52100E+03	0.00000E+00	0.00000E+00	0.85000E+02	
MEDIUM	2	0.89000E+02	0.23000E+03	0.10000E+02	0.15000E+02	0.64000E+02	
--> NEGEX=	35	LOWAZ= 27	NWTC= 4				
/SUBMIT(1):	NEUTRONS:114132, PIO: 1136, NUCLEI: 16528						

MEDIUM	SOURCE= FRACTION	%ERR			
1	1.000E+00	0.00			
2	0.000E+00	0.00			
VOID	0.000E+00	0.00			
MEDIUM	INELASTIC C./SOURCE	%ERR	ELASTIC C. /SOURCE	%ERR	
1	1.160E+01	1.41	1.803E+01	1.65	
2	3.298E+00	2.11	2.847E+00	2.42	
VOID	0.000E+00	0.00	0.000E+00	0.00	
MEDIUM	ABSORPTION /SOURCE	%ERR	FISSION /SOURCE	%ERR	
1	3.474E+00	2.06	6.965E+00	1.52	
2	1.150E-01	9.44	0.000E+00	0.00	
VOID	0.000E+00	0.00	0.000E+00	0.00	
MEDIUM	P FLUX (CM)	%ERR	N FLUX (CM)	%ERR	
1	1.165E+01	2.68	1.550E+02	1.59	
2	9.859E+00	2.66	1.283E+02	1.59	
VOID	1.954E+00	2.70	2.568E+01	1.60	
MEDIUM	PI+ FLUX (CM)	%ERR	PI+ FLUX (CM)	%ERR	
1	4.166E+00	5.89	1.915E+01	2.26	
2	3.478E+00	5.90	1.581E+01	2.28	
VOID	6.936E-01	5.89	3.161E+00	2.27	
MEDIUM	MU+ FLUX (CM)	%ERR	MU- FLUX (CM)	%ERR	
1	3.633E-02	98.78	7.024E-03	87.26	
2	2.987E-02	99.23	8.303E-04	100.00	
VOID	6.348E-03	93.58	1.661E-04	100.00	
MEDIUM	P YIELD	%ERR	N YIELD	%ERR	
1	1.773E+01	1.55	1.487E+02	1.29	
2	3.882E+00	2.49	4.918E+00	2.43	
VOID	0.000E+00	0.00	0.000E+00	0.00	
MEDIUM	PI+ YIELD	%ERR	PI0 YIELD	%ERR	
1	8.671E-01	4.35	1.404E+00	2.98	
2	1.159E-01	10.78	1.361E-01	10.74	
VOID	0.000E+00	0.00	0.000E+00	0.00	
MEDIUM	PI- YIELD	%ERR			
1	2.027E+00	2.70			
2	2.061E-01	8.42			
VOID	0.000E+00	0.00			
MEDIUM	MU+ YIELD	%ERR	MU- YIELD	%ERR	
1	1.093E-03	100.00	2.053E-03	70.68	
2	9.814E-04	100.00	0.000E+00	0.00	
VOID	0.000E+00	0.00	0.000E+00	0.00	
MEDIUM	D YIELD	%ERR	T YIELD	%ERR	
1	3.821E+00	2.66	4.217E+00	2.37	
2	6.372E-02	14.75	1.018E-02	33.42	
VOID	0.000E+00	0.00	0.000E+00	0.00	
MEDIUM	HE3 YIELD	%ERR	HE4 YIELD	%ERR	
1	4.940E-01	6.68	8.091E+00	2.42	
2	1.386E-02	34.17	1.749E+00	4.39	
VOID	0.000E+00	0.00	0.000E+00	0.00	

W:RSYS: NO RSYST-TAPE AVAILABLE

VISIBLE ENERGY DEPOSITION

DETECTOR NAME = EVIS

DEPOSITION-TYPE= P

REGION	ENERGY	%ERR
1001000	1.457E+01	5.411
1002000	1.678E+01	4.327
1003000	1.184E+01	5.053
1004000	7.719E+00	6.175
1005000	5.026E+00	7.894
1006000	3.029E+00	10.484
1007000	1.538E+00	13.804
2001000	8.957E-01	13.449
2002000	1.816E+00	9.900
2003000	2.824E+00	7.871
2004000	2.996E+00	7.359
2005000	2.663E+00	7.943
2006000	1.808E+00	9.862
2007000	1.187E+00	12.326

REGION	ENERGY	%ERR
1001000	1.515E+00	8.656
1002000	2.039E+00	7.528
1003000	1.628E+00	8.919
1004000	9.604E-01	10.974
1005000	6.260E-01	12.910
1006000	4.107E-01	17.012
1007000	2.009E-01	20.560
2001000	3.357E-02	51.677
2002000	1.885E-01	26.137
2003000	3.845E-01	20.482
2004000	3.469E-01	21.042
2005000	2.515E-01	24.135
2006000	1.331E-01	29.676
2007000	1.083E-01	37.290

REGION	ENERGY	%ERR
1001000	1.194E+01	1.299
1002000	9.155E+00	2.454
1003000	6.180E+00	3.697

ENERGY DEPOSITIED

DETECTOR NAME = EDEP

DEPOSITION-TYPE	ENERGY	%ERR
P	8.742E+02	1.715
PI+	1.163E+02	5.488
PI-	5.595E+02	2.134
MU+	1.068E+00	94.598
MU-	2.733E-01	76.539
D	4.305E+01	3.075
T	4.723E+01	2.683
HE3	9.373E+00	7.146
ALPHA	1.597E+02	2.300
RECOIL	1.914E+03	1.537
P LEAK	3.875E+01	13.872
N LEAK	3.965E+02	3.062
PI+ LEAK	2.951E+01	19.617
PI- LEAK	1.236E+02	14.707
MU+ LEAK	1.299E+00	100.000
MU- LEAK	0.000E+00	0.000
N SUBMIT	4.423E+02	1.294
PI0 SUBMIT	8.861E+02	3.738
EXI. SUBMIT	1.124E+02	1.422

ENERGY ON SURFACE

DETECTOR NAME = ELOS

PARTICLE-TYPE= p	REGION1	REGION2	ENERGY	%ERR
	0	3	3.663E+00	34.460
	0	4	1.006E+01	16.121
	0	5	2.503E+01	20.071

PARTICLE-TYPE= n	REGION1	REGION2	ENERGY	%ERR
	0	3	1.616E+02	4.099
	0	4	1.075E+02	5.486
	0	5	1.273E+02	7.871

I:EXIT;

HERMES TEST1 14 DETEKTOREN CARD A

NSTRY	NMOST	NITS	NQUIT	NGPQTW	NGPQTC	NHGP	NMTG	NCOLTP	IADJM	MAXTIM	MEDIA	MEDALB
113	2500	450	1	100	21	100	121	0	0	58.00	3	0

ISOUR	NGPFS	ISBIAS	WTSTRT	EBOTW	EBOTG	TCUT	VELTH
1	1	0	1.0000E+00	1.0000E-04	1.0000E+04	1.0000E-07	6.2930E+05

XSTRT	YSTRT	ZSTRT	AGSTRT	UINP	VINP	WINP
0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000000	0.000000	0.000000

***UNIT-NUMBER OF HERMES SUBMISSION : 50

GROUP PARAMETERS, GROUP NUMBERS GREATER THAN 100 CORRESPOND TO SECONDARY PARTICLES

GROUP	UPPER EDGE (EV)	VELOCITY (CM/SEC)
1	1.4900E+07	5.2115E+09
2	1.3500E+07	4.9576E+09
:		
120	2.0000E+05	2.9979E+10
121	1.0000E+05	2.9979E+10

INITIAL RANDOM NUMBER = 35FA731A

NSPLT= 1 NKILL= 1 NPAST= 0 NOLEAK= 0 IEBIAS= 0 MXREG= 3 MAXGP=121

WEIGHT STANDARDS FOR SPLITTING AND RUSSIAN ROULETTE AND PATHLENGTH STRETCHING PARAMETERS

NGP1	NDG	NGP2	NRG1	NDRC	NRG2	WTHIN1	WTLOW1	WTAVE1	XNU
0	0	0	0	0	0	1.0000E+00	1.0000E+00	1.0000E+00	0.0000E+00

NSOUR= 0 NFISTP= 1 NKCALC= 0 NORMF= 0

FWLOW= 1.00000E+00, 1.00000E+00, 1.00000E+00,

SOURCE DISTRIBUTION FOR FISSION NEUTRONS

GROUP	IMED= 1	IMED= 2	IMED= 3
1	4.1220E-05	0.0000E+00	4.1220E-05
2	1.1330E-04	0.0000E+00	1.1330E-04
:			
120	0.0000E+00	0.0000E+00	0.0000E+00
121	0.0000E+00	0.0000E+00	0.0000E+00

GWLOW(IG,MREG)

GROUP	REGION 1	REGION 2	REGION 3
1	1.0000E+00	1.0000E+00	1.0000E+00
2	1.0000E+00	1.0000E+00	1.0000E+00
:			
98	1.0000E+00	1.0000E+00	1.0000E+00
99	1.0000E+00	1.0000E+00	1.0000E+00
100	1.0000E+00	1.0000E+00	1.0000E+00

NUMB. OF BODIES (NUMB) 412
 NUMB. OF REGIONS (NUMR) 435
 NUMBER OF CODE REGIONS (NCOD) 435
 LENGTH OF FDP ARRAY (LMDX) 4600
 LENGTH OF MA ARRAY (LMYY) 16000
 MODIFIED INPUT (LOUT) 0
 VOLUME INPUT OPTION (IVOPT) 4 NOT USED
 BOUNDARY OPTION (MEDREG) 0

(3) Output of MORSE

DEBUG OPTION (IDBG0)

0

MACHINE EPS = 0.100000E-07

IPRINT= 0
STRETCH= 1.000000E+00

NUMBER OF BODIES 412
LENGTH OF MA -ARRAY 2885
LENGTH OF FPD-ARRAY 4543
SUB-GEOMETRY #2
NUMBER OF INPUT ZONES 414
NUMBER OF CODE ZONES 414
LENGTH OF INTEGER ARRAY 6619
NGEOM= 10295, NGLAST= 29103

NUMBER OF PRIMARY GROUPS (NGP) 100
 NUMBER OF PRIMARY DOWNSCATTERS (NDS) 100
 NUMBER OF SECONDARY GROUPS (NGG) 21
 NUMBER OF SECONDARY DOWNSCATTERS (NDSC) 21
 NUMBER OF PRIM*SEC GROUPS (INGP) 121
 TABLE LENGTH (ITBL) 124
 LOC OF WITHIN GROUP (SIG GG) (ISGG) 4
 NUMBER OF MEDIA (NMED) 3
 NUMBER OF INPUT ELEMENTS (NELEM) 4
 NUMBER OF MIXING ENTRIES (NMIX) 5
 NUMBER OF COEFFICIENTS (NCOEF) 6
 NUMBER OF ANGLES (NSCT) 3
 RESTORE COEFF (ISTAT) 0
 ADJOINT SWITCH (FROM MORSE) 0

INPUT/OUTPUT OPTIONS
 IRDSG (AS READ) 0
 ISTR (AS STORE) 0
 IFMU (MUS) 0
 IMOM (MOMENTS) 0
 IPRIN (ANGLES, PROB) 0
 IPUN (IMPOSSIBLE COEF) 0
 CARD FORMAT (IDTF) 0
 INPUT TAPE (IXTAPE) 20
 MORSEC TAPE (JXTAPE) 0
 OGR TAPE (IOGRT) 0
 ICOPY (LAST GRP, OGR) 0
 CROSS-TAPE AUS RSYST 0
 ICRO CROSS. FOR MEDIA
 N TO C TRANSFER 0
 PRINT OF BADMOMENTS 0

ELEMENTS FROM LIBRARY TAPE		IDENTIFIERS														
		64	65	66	67	68	69	127	128	129	130	131	132	262	263	
ELEMENT	1	264	265	266	267	253	254	255	256	257	258					
ELEMENT	2				64		121	124	0	64C	1274	100N-21G	GPS	P8	800K	07/25/84
ELEMENT	3				127		121	124	0	127H	1269	100N-21G	GPS	P8	800K	WATER BOUND 07/25/84
ELEMENT	4				262		121	124	0	262	P0	U-238	1262	100N-21G	P8	800K MOD 17 INF DIL 07/25/84
ELEMENT					253		121	124	0	253U-235	1261	100N-21G	GPS	P8	800K	INF DIL 07/25/84

STORAGE ALLOCATIONS
 CROSS SECTIONS START AT 29104
 LAST LOCATION USED (PERM) 148701
 TEMP LOCATIONS USED 784848 TO 950000
 EXCESS STORAGE (TEMP) 636147

ELEMENT	1	64	121	124	0	64C	1274	100N-21G	GPS	P8	800K	07/25/84
ELEMENT	1	65	121	124	0	65C	1274	100N-21G	GPS	P8	800K	07/25/84
ELEMENT	1	66	121	124	0	66C	1274	100N-21G	GPS	P8	800K	07/25/84
ELEMENT	1	67	121	124	0	67C	1274	100N-21G	GPS	P8	800K	07/25/84
ELEMENT	1	68	121	124	0	68C	1274	100N-21G	GPS	P8	800K	07/25/84
ELEMENT	1	69	121	124	0	69C	1274	100N-21G	GPS	P8	800K	07/25/84
ELEMENT	2	127	121	124	0	127H	1269	100N-21G	GPS	P8	800K	WATER BOUND 07/25/84
ELEMENT	2	128	121	124	0	128H	1269	100N-21G	GPS	P8	800K	WATER BOUND 07/25/84
ELEMENT	2	129	121	124	0	129H	1269	100N-21G	GPS	P8	800K	WATER BOUND 07/25/84
ELEMENT	2	130	121	124	0	130H	1269	100N-21G	GPS	P8	800K	WATER BOUND 07/25/84
ELEMENT	2	131	121	124	0	131H	1269	100N-21G	GPS	P8	800K	WATER BOUND 07/25/84
ELEMENT	2	132	121	124	0	132H	1269	100N-21G	GPS	P8	800K	WATER BOUND 07/25/84
ELEMENT	3	262	121	124	0	262	P0	U-238	1262	100N-21G	P8	800K MOD 17 INF DIL 07/25/84
ELEMENT	3	263	121	124	0	263	P1	U-238	1262	100N-21G	P8	800K MOD 17 INF DIL 07/25/84
ELEMENT	3	264	121	124	0	264	P2	U-238	1262	100N-21G	P8	800K MOD 17 INF DIL 07/25/84
ELEMENT	3	265	121	124	0	265	P3	U-238	1262	100N-21G	P8	800K MOD 17 INF DIL 07/25/84
ELEMENT	3	266	121	124	0	266	P4	U-238	1262	100N-21G	P8	800K MOD 17 INF DIL 07/25/84
ELEMENT	3	267	121	124	0	267	P5	U-238	1262	100N-21G	P8	800K MOD 17 INF DIL 07/25/84
ELEMENT	4	253	121	124	0	253U-235	1261	100N-21G	GPS	P8	800K	INF DIL 07/25/84

ELEMENT	4	254	121	124	0	254U-235	1261	100N-21G	GPS	P8	800K	INF	DIL	07/25/84
ELEMENT	4	255	121	124	0	255U-235	1261	100N-21G	GPS	P8	800K	INF	DIL	07/25/84
ELEMENT	4	256	121	124	0	256U-235	1261	100N-21G	GPS	P8	800K	INF	DIL	07/25/84
ELEMENT	4	257	121	124	0	257U-235	1261	100N-21G	GPS	P8	800K	INF	DIL	07/25/84
ELEMENT	4	258	121	124	0	258U-235	1261	100N-21G	GPS	P8	800K	INF	DIL	07/25/84

MIXING TABLE

MEDIA	1	CONTAINS ELEMENT	3	WITH DENSITY	4.7932E-02
MEDIA	1	CONTAINS ELEMENT	4	WITH DENSITY	1.4607E-04
MEDIA	2	CONTAINS ELEMENT	2	WITH DENSITY	4.8295E-02
MEDIA	2	CONTAINS ELEMENT	1	WITH DENSITY	4.8295E-02
MEDIA	3	CONTAINS ELEMENT	2	WITH DENSITY	4.8295E-02

BANKS START AT 148702
LAST LOCATION USED 186201

ANALYSIS INPUT

CARD AA

ND= 14, NME=100, NE=121, NT= 0, NA= 0, NRESP= 1, NEX= 0, NEXND=100

IORESP=1 IOFLUX=1

DET X Y Z MED1 MED2

TO IS SET TO 0.0 IN ALL CASES

1	1001000	0.0000E+00	0.0000E+00
2	1002000	0.0000E+00	0.0000E+00
3	1003000	0.0000E+00	0.0000E+00
4	1004000	0.0000E+00	0.0000E+00
5	1005000	0.0000E+00	0.0000E+00
6	1006000	0.0000E+00	0.0000E+00
7	1007000	0.0000E+00	0.0000E+00
8	2001000	0.0000E+00	0.0000E+00
9	2002000	0.0000E+00	0.0000E+00
10	2003000	0.0000E+00	0.0000E+00
11	2004000	0.0000E+00	0.0000E+00
12	2005000	0.0000E+00	0.0000E+00
13	2006000	0.0000E+00	0.0000E+00
14	2007000	0.0000E+00	0.0000E+00

GROUP	RESP(1)
1	1.0000E+00
2	1.0000E+00

120	0.0000E+00
121	0.0000E+00

BIN NO.	LOWER LIMIT GROUP	LOWER ENERGY LIMIT	DELTA E
1	1	1.490E+07	1.400E+06
2	2	1.350E+07	1.300E+06

120	120	1.000E+05	1.000E+05
121	121	1.000E+04	9.000E+04

NUMBER OF TIME BINS 0

NUMBER OF ANGLE BINS 0
UPPER LIMITS OF COSINE BINS

7375 CELLS USED BY ANALYSIS, 756424 CELLS REMAIN UNUSED.

DETECTOR NO	REGION NO	1001000	NEL = 1	DENH = 0.4829500E-01	S = 1.00	KB = 0.8499999E-02
1	Z = 6.0	A = 12.0	DEN = 0.4829484E-01			
2	REGION NO	1002000	NEL = 1	DENH = 0.4829500E-01	S = 1.00	KB = 0.8499999E-02
	Z = 6.0	A = 12.0	DEN = 0.4829484E-01			
13	REGION NO	2006000	NEL = 1	DENH = 0.4829500E-01	S = 1.00	KB = 0.8499999E-02
	Z = 6.0	A = 12.0	DEN = 0.4829484E-01			

TIME REQUIRED FOR INPUT WAS 3 MINUTES 51 SECONDS .
YOU ARE USING THE DEFAULT VERSION OF STRUN WHICH DOES NOTHING.

***START BATCH 1 RANDOM= 35FA731A

SOURCE DATA

HET-IDENTIFIKATION: REWO 31N1 05/0 3/88 12:3 6:16

YOU ARE USING THE DEFAULT VERSION OF SOURCE WHICH SETS WATE TO DDF AND PROVIDES AN ENERGY IG.

WTAVE	IAVE	UAVE	VAVE	WAVE	XAVE	YAVE	ZAVE	AGEAVE
1.804E+02	19.25	0.0314	0.0012	-0.0682	-3.824E+00	8.615E-01	3.818E+01	0.000E+00

NUMBER OF COLLISIONS OF TYPE NCOLL

SOURCE SPLIT(D)	FISHN	GAMGEN	REALCOLL	ALBEDO	BDRYX	ESCAPE	E-CUT	TIMEKILL	R R	KILL	R R	SURV	GAMLOST	FISGEN
156	863	97	971	5494	0	23706	22	0	191	1874	3703	0	97	

TIME REQUIRED FOR THE PRECEDING BATCH WAS 22 SECONDS .

***START BATCH 2 RANDOM= 696C252A

SOURCE DATA

WTAVE	IAVE	UAVE	VAVE	WAVE	XAVE	YAVE	ZAVE	AGEAVE
1.021E+02	20.15	0.0721	-0.0215	0.0527	-1.233E+00	-3.514E-01	4.547E+01	0.000E+00

NUMBER OF COLLISIONS OF TYPE NCOLL

SOURCE SPLIT(D)	FISHN	GAMGEN	REALCOLL	ALBEDO	BDRYX	ESCAPE	E-CUT	TIMEKILL	R R	KILL	R R	SURV	GAMLOST	FISGEN
102	416	43	429	2572	0	11217	20	0	100	872	1783	0	140	

TIME REQUIRED FOR THE PRECEDING BATCH WAS 8 SECONDS .

***START BATCH 450 RANDOM= 6278807E

SOURCE DATA

WTAVE	IAVE	UAVE	VAVE	WAVE	XAVE	YAVE	ZAVE	AGEAVE
1.832E+02	21.51	0.0559	0.0260	-0.0038	4.764E-01	2.013E+00	4.309E+01	0.000E+00

NUMBER OF COLLISIONS OF TYPE NCOLL

SOURCE SPLIT(D)	FISHN	GAMGEN	REALCOLL	ALBEDO	BDRYX	ESCAPE	E-CUT	TIMEKILL	R R	KILL	R R	SURV	GAMLOST	FISGEN
157	763	62	792	4652	0	19248	23	0	165	1586	3159	0	25647	

TIME REQUIRED FOR THE PRECEDING BATCH WAS 12 SECONDS .

/ VISIBLE ENERGY IN SCSN38 ; 7X INNER, 7X OUTER, 7X OUTER TOWER EXTRA
RESPONSES(DETECTOR)

DETECTOR	UNCOLL RESPONSE	FSD UNCOLL	TOTAL RESPONSE	FSD TOTAL
1	0.0000E+00	0.00000	5.5550E+00	0.05651
2	0.0000E+00	0.00000	6.6532E+00	0.05022
3	0.0000E+00	0.00000	4.9863E+00	0.05195
4	0.0000E+00	0.00000	3.5615E+00	0.06393
5	0.0000E+00	0.00000	2.3326E+00	0.08263
6	0.0000E+00	0.00000	1.2973E+00	0.10683
7	0.0000E+00	0.00000	5.1285E-01	0.15913
8	0.0000E+00	0.00000	2.3704E+00	0.05214
9	0.0000E+00	0.00000	3.7826E+00	0.04301
10	0.0000E+00	0.00000	4.0207E+00	0.03782
11	0.0000E+00	0.00000	3.4266E+00	0.04421
12	0.0000E+00	0.00000	2.5918E+00	0.05474
13	0.0000E+00	0.00000	1.8419E+00	0.06765
14	0.0000E+00	0.00000	9.0301E-01	0.10086

CARD DD

DETECTOR NO. ENERGIES	1	2	3	4	5	6	7	8	9	10	CARD GG
1.490E+07	7.037E-08	1.407E-07	1.056E-07	5.278E-08	3.518E-08	2.346E-08	5.864E-09	2.932E-08	4.691E-08	5.864E-08	
	0.309	0.246	0.257	0.366	0.470	0.498	1.000	0.445	0.351	0.344	
1.350E+07	1.162E-07	1.162E-07	6.639E-08	6.639E-08	4.426E-08	5.533E-09	0.000E+00	2.213E-08	6.639E-08	7.192E-08	
	0.224	0.224	0.309	0.309	0.351	1.000	0.000	0.498	0.285	0.274	
1.220E+07	9.177E-08	1.606E-07	9.751E-08	7.457E-08	6.883E-08	1.147E-08	1.147E-08	5.162E-08	5.162E-08	6.883E-08	
	0.261	0.183	0.266	0.350	0.330	0.706	0.706	0.330	0.398	0.285	
1.110E+07	1.456E-07	1.707E-07	9.537E-08	1.154E-07	6.023E-08	3.012E-08	3.012E-08	5.019E-08	8.031E-08	7.027E-08	
	0.193	0.170	0.237	0.221	0.309	0.406	0.406	0.344	0.276	0.263	

DETECTOR NO. ENERGIES	11	12	13	14						CARD GG
2.000E+05	0.000E+00	0.000E+00	0.000E+00	0.000E+00						
	0.000	0.000	0.000	0.000						
1.000E+05	0.000E+00	0.000E+00	0.000E+00	0.000E+00						
	0.000	0.000	0.000	0.000						
1.000E+04										
EXTRA ARRAYS OF LENGTH ND										
EXT1(14)										
(1)	0.369429E+01	0.369429E+01	0.369429E+01	0.369429E+01	0.369429E+01	0.369429E+01	0.369429E+01	0.369429E+01	0.369429E+01	0.369429E+01
(9)	0.369429E+01	0.369429E+01	0.369429E+01	0.369429E+01	0.369429E+01	0.369429E+01	0.369429E+01	0.369429E+01	0.369429E+01	0.369429E+01
EXT2(14)										
(1)	0.323659E+01	0.323659E+01	0.323659E+01	0.323659E+01	0.323659E+01	0.323659E+01	0.323659E+01	0.323659E+01	0.323659E+01	0.323659E+01
(9)	0.323659E+01	0.323659E+01	0.323659E+01	0.323659E+01	0.323659E+01	0.323659E+01	0.323659E+01	0.323659E+01	0.323659E+01	0.323659E+01
E100(14)										
(1)	0.883867E-07	0.883867E-07	0.883867E-07	0.883867E-07	0.883867E-07	0.883867E-07	0.883867E-07	0.883867E-07	0.883867E-07	0.883867E-07
(9)	0.883867E-07	0.883867E-07	0.883867E-07	0.883867E-07	0.883867E-07	0.883867E-07	0.883867E-07	0.883867E-07	0.883867E-07	0.883867E-07

TIME REQUIRED FOR THE PRECEDING 450 BATCHES WAS 1 HOUR 17 MINUTES 13 SECONDS .

NEUTRON DEATHS	NUMBER	WEIGHT
KILLED BY RUSSIAN ROULETTE	527385	3.12670E+05
ESCAPED	13776	1.37760E+04
REACHED ENERGY CUTOFF	0	0.00000E+00
REACHED TIME CUTOFF	53040	5.30400E+04

** NEXT RANDOM NUMBER IS 3CB08A3E

TOTAL CPU TIME FOR THIS PROBLEM WAS 73.38 MINUTES.

(4) No Output of NDEM

(5) Output of PEGS on Tape

(6) Output of EGS

/JOB-ID = REW03102 05/09/88 08:32:18

IRUN= 1 MHIS= 1 MBAT= 500 NSKIP= 0
INEGS= 1 IOEGS= 2 INGEO= 5 IOGEO= 6 KMPI= 12 KMPO= 8

IXX= 123456789
IQI= -1 EI= 5.000E+03 WT1= 1.000E+00
XI= 0.000E+00 YI= 0.000E+00 ZI= 1.000E-02
UI= 0.000E+00 VI= 0.000E+00 WI= 1.000E+00

NMED= 2 NREG= 19
EGS SUCCESSFULLY 'MATCHED' FOR 2 MEDIA.

----- MEDNAM -----		ESTEP	AE	AP
URAN		5.000E-03	7.110E-01	1.000E-01
SCSN38		1.000E-02	7.110E-01	1.000E-01
REGION	LSTREG	MED	ECUT	PCUT
1	1001000	2	1.511E+00	1.000E-01
2	1002000	2	1.511E+00	1.000E-01
3	1003000	2	1.511E+00	1.000E-01
4	1004000	2	1.511E+00	1.000E-01
5	1005000	2	1.511E+00	1.000E-01
6	1006000	2	1.511E+00	1.000E-01
7	1007000	2	1.511E+00	1.000E-01
8	2001000	2	1.511E+00	1.000E-01
9	2002000	2	1.511E+00	1.000E-01
10	2003000	2	1.511E+00	1.000E-01
11	2004000	2	1.511E+00	1.000E-01
12	2005000	2	1.511E+00	1.000E-01
13	2006000	2	1.511E+00	1.000E-01
14	2007000	2	1.511E+00	1.000E-01
15	1	1	1.511E+00	1.000E-01
16	2	0	0.000E+00	0.000E+00
17	3	0	0.000E+00	0.000E+00
18	4	0	0.000E+00	0.000E+00
19	5	0	0.000E+00	0.000E+00

NUMB. OF BODIES (NUMB) 412
NUMB. OF REGIONS (NUMR) 435
NUMBER OF CODE REGIONS (NCOD) 435
LENGTH OF FDP ARRAY (LMXX) 4600
LENGTH OF MA ARRAY (LMYY) 16000
MODIFIED INPUT (LOUT) 0
VOLUME INPUT OPTION (IVOPT) 4 NOT USED
BOUNDARY OPTION (MEDREG) 0
DEBUG OPTION (IDBGO) 0

MACHINE EPS = 0.100000E-07

IPRINT= 0
STRETCH= 1.000000E+00

NUMBER OF BODIES 412
LENGTH OF MA -ARRAY 2885
LENGTH OF FDP-ARRAY 4543
SUB-GEOMETRY #2
NUMBER OF INPUT ZONES 414
NUMBER OF CODE ZONES 414
LENGTH OF INTEGER ARRAY 6619
EOB EGS 1 1 1 TBAT= 5.805 IXX= -1527888639
EOB EGS 1 2 1 TBAT= 23.438 IXX= 47469917

EOB EGS 1 499 1 TBAT= 9.559 IXX= -56670779
EOB EGS 1 500 1 TBAT= 0.098 IXX= -56670779
ENERGY DEPOSITION IN MEV WITH IARG=0

EGS-DET	REGION	E-	%ERR	GAMMA	%ERR	E+	%ERR	SUMME	%ERR
1	1001000	4.240E+00	12.36	0.000E+00	0.00	3.050E+00	11.94	7.290E+00	11.88

2	1002000	4.782E+00	10.75	0.000E+00	0.00	3.095E+00	11.22	7.877E+00	10.78
3	1003000	3.281E+00	12.94	0.000E+00	0.00	2.288E+00	14.01	5.569E+00	13.06
4	1004000	2.054E+00	17.50	0.000E+00	0.00	1.609E+00	18.16	3.663E+00	17.29
5	1005000	1.208E+00	23.15	0.000E+00	0.00	9.260E-01	25.32	2.134E+00	23.66
6	1006000	4.585E-01	27.54	0.000E+00	0.00	3.210E-01	27.82	7.795E-01	27.25
7	1007000	3.697E-01	39.54	0.000E+00	0.00	2.263E-01	39.17	5.960E-01	39.28
8	2001000	2.086E-02	33.85	0.000E+00	0.00	2.452E-02	67.01	4.537E-02	46.21
9	2002000	7.859E-02	42.23	0.000E+00	0.00	2.128E-02	62.82	9.987E-02	45.90
10	2003000	4.133E-02	46.79	0.000E+00	0.00	1.341E-02	58.56	5.474E-02	46.33
11	2004000	1.224E-01	42.94	0.000E+00	0.00	1.282E-01	40.80	2.506E-01	40.21
12	2005000	1.428E-01	33.54	0.000E+00	0.00	6.511E-02	33.40	2.079E-01	31.10
13	2006000	1.025E-01	32.77	0.000E+00	0.00	1.080E-01	39.36	2.105E-01	39.31
14	2007000	5.358E-02	37.07	0.000E+00	0.00	2.314E-02	64.41	7.672E-02	52.76
15	1	2.607E+02	5.26	0.000E+00	0.00	1.751E+02	5.29	4.354E+02	5.25
16	2	0.000E+00	0.00	0.000E+00	0.00	0.000E+00	0.00	0.000E+00	0.00
17	3	0.000E+00	0.00	0.000E+00	0.00	0.000E+00	0.00	0.000E+00	0.00
18	4	0.000E+00	0.00	0.000E+00	0.00	0.000E+00	0.00	0.000E+00	0.00
19	5	0.000E+00	0.00	0.000E+00	0.00	0.000E+00	0.00	0.000E+00	0.00
SUMME		2.776E+02	5.26	0.000E+00	0.00	1.870E+02	5.28	4.641E+02	5.25
.									
IARG	ALL REGIONS	E-	%ERR	GAMMA	%ERR	E+	%ERR	SUMME	%ERR
0		2.776E+02	5.26	0.000E+00	0.00	1.870E+02	5.28	4.641E+02	5.25
1		2.983E+02	5.29	0.000E+00	0.00	4.512E+01	5.32	3.434E+02	5.29
2		2.604E+01	5.33	1.189E-01	6.85	1.436E-01	7.20	2.630E+01	5.33
3		1.222E+00	36.10	1.289E+01	23.69	1.050E+00	43.79	1.516E+01	22.52
4		0.000E+00	0.00	4.846E+01	5.30	0.000E+00	0.00	4.846E+01	5.30
FREQUENCY OF IARG=0									
EGS-DET	REGION	E-	%ERR	GAMMA	%ERR	E+	%ERR	SUMME	%ERR
1	1001000	1.165E+02	12.17	1.405E+02	12.43	6.784E+01	12.29	3.248E+02	11.99
.									
19	5	0.000E+00	0.00	0.000E+00	0.00	0.000E+00	0.00	0.000E+00	0.00
SUMME		1.680E+04	5.28	2.490E+03	5.64	8.473E+03	5.31	2.767E+04	5.30
.									
IARG	ALL REGIONS	E-	%ERR	GAMMA	%ERR	E+	%ERR	SUMME	%ERR
0		1.680E+04	5.28	2.490E+03	5.64	8.473E+03	5.31	2.767E+04	5.30
1		5.539E+02	5.29	0.000E+00	0.00	5.099E+01	5.31	6.048E+02	5.29
2		2.847E+02	5.32	1.479E+01	5.51	2.625E+00	6.09	3.021E+02	5.32
3		7.747E-02	26.33	2.128E+00	16.72	5.316E-02	27.01	2.259E+00	16.81
4		0.000E+00	0.00	4.201E+02	5.30	0.000E+00	0.00	4.201E+02	5.30
FLUX IN CM									
EGS-DET	REGION	E-	%ERR	GAMMA	%ERR	E+	%ERR	SUMME	%ERR
1	1001000	2.470E+00	12.36	5.605E+01	11.79	1.785E+00	11.94		
.									
19	5	0.000E+00	0.00	0.000E+00	0.00	0.000E+00	0.00		
LEAKAGE CURRENT									
EGS-DET	REGION	E-	%ERR	GAMMA	%ERR	E+	%ERR	SUMME	%ERR
1	1001000	0.000E+00	0.00	0.000E+00	0.00	0.000E+00	0.00		
.									
19	5	1.251E-02	57.63	3.156E-01	45.16	4.228E-03	70.64		
EOR EGS 1 500 500 REM03102 05/09/88 08:32:18									
TOTAL TIME USED =*****									
MULTIPLE SCATTERING PER EVENT									
TRIED FAILED FRACTION FAILED[%]									
21251.11 3.98 0.02									

(7) Output of STATIST

```

I:/ Test1 STATIST
I:/ 1) define data arrays for STATIST analysis
I:/ 1.1) setup arrays to read in HET, MORSE, and EGS(pi0) detector records
I: REAL EDEP(19);
I: REAL FHET(14,19), FMOR(14), FPIO(19,3,5);
I:/ 1.2) setup arrays to keep the sums
I:/ over all contribution types of interest
I: REAL EHET(14), EMOR(14), EPIO(19);
I:/ 1.3) setup weight arrays used to extract the contributions of interest
I: REAL WHET(19), WMOR(1), WPIO(15);
I: DATA WHET=< 1. 1. 1. F 0.>;
I: DATA WMOR=1.;
I: DATA WPIO=< R 3 1. R 3 1. R 3 1. R 3 0. 0. 1. 0.>;
I:/ 1.4) setup an array to sum over all scintillator regions
I: REAL SCINT(19); DATA SCINT=<R 14 1. F 0.>;
I:/ 1.5) setup variables for final event results
I: REAL EVCHG(1), EVPIO(1), EVNEU(1);
I:/ 2) start event processing (7 variables 450 events)
I: STATIN 7,450;
I:/ 2.1) load detector records from submission files
I: LOAD 10,EVIS@EOB,FHET; / HET DIFFERENTIAL PULSE DISTRIBUTION
I: LOAD 10,EDEP@EOB,EDEP; / HET GLOBAL ENERGY DEPOSITION
I: LOAD 11,R001@EOB,FMOR; / MORSE VISIBLE N ENERGY
I: LOAD 12,EDEP@EOB,FPIO; / EGS VISIBLE P10 ENERGY
I:/ 2.2) extract main contributions
I: DOT EHET=EHET*WHET; / EXTRACT EVIS
I: DOT EMOR=FMOR*WMOR; / EXTRACT EVIS
I: DOT EPIO=FPIO*WPIO (19,15,1); / EXTRACT EVIS
I:/ 2.3) sum over all scintillator regions
I: DOT EVCHG=EHET*SCINT (1,14,1);
I: DOT EVNEU=EMOR*SCINT (1,14,1);
I: DOT EVPIO=EPIO*SCINT (1,19,1);
I:
I:/ 2.4) assign V containing 7 variables
I:/ for each event, V is stacked into EVENT
I: SET V( 1)=EDEP(1)+EDEP(2)+EDEP(3);
I: SET V( 2)=EDEP(17);
I: SET V( 3)=EDEP(18);
I: SET V( 4)=EVCHG(1);
I: SET V( 5)=EVNEU(1);
I: SET V( 6)=EVPIO(1);
I: SET V( 7)=V(4)+V(5)+V(6);
I:/ 3) start end of run analysis
I: BIN 1,0.,5000., 50, ' Edep p,pi+,pi-';
I: BIN 2,0.,1000., 50, ' Esub n';
I: BIN 3,0.,5000., 50, ' Esub p10';
I:
I: BIN 4,0.,500.0, 50, ' Evis p,pi+,pi-';
I: BIN 5,0.,100.0, 50, ' Evis n';
I: BIN 6,0.,500.0, 50, ' Evis p10';
I:
I: BIN 7,0.,500.0, 50, ' Evis p,pi+,pi-,n,p10';
I: / NOTE: FIT and SCAT require a previous call of BIN
I: / for the variables wanted.
I: FIT 7;
I: SCAT 1,4;
I: SCAT 2,5;
I: SCAT 3,6;
I: STATIST PLOT;

```

/JOB-ID = REW03112 05/26/88 09:27:47

SOR HET 1 0 0 REW031H1 05/03/88 12:36:16

PGM MOR REW10102 05/17/88

gC

SOR HET 1 0 0 REW031H1 05/03/88 12:36:16

BI-VARIATE MOMENTS STATIST REW03112 05/26/88 09:27:47

VARIABLE	MEAN(X)	STD(X)	COV(X,Y)	A
Y	MEAN(Y)	STD(Y)	RHO(X,Y)	B

Edep p,pi+,pi-	1.542E+03	5.538E+02	2.310E+04	7.534E-02
Evis p,pi+,pi-	1.214E+02	4.575E+01	9.119E-01	5.225E+00

Esub n	4.530E+02	1.705E+02	2.764E+03	9.503E-02
Evis n	4.384E+01	1.797E+01	9.017E-01	7.834E-01

Esub pi0	8.984E+02	1.032E+03	5.041E+04	4.730E-02
Evis pi0	4.179E+01	5.004E+01	9.759E-01	7.079E-01

UNIVARIATE DATA

VARIABLE	N-EVENTS	N-ZERO	N-LOST	MINIMUM	MAXIMUM
Edep p,pi+,pi-	450.	0.	0.	1.949E+02	3.154E+03
Esub n	450.	2.	0.	9.265E+00	9.338E+02
Esub pi0	450.	117.	0.	1.394E+02	4.715E+03
Evis p,pi+,pi-	450.	0.	0.	1.291E+01	2.382E+02
Evis n	450.	2.	0.	3.501E-01	9.972E+01
Evis pi0	450.	118.	0.	1.809E+00	2.120E+02
Evis p,pi+,pi-,n,pi0	450.	0.	0.	5.315E+01	3.310E+02

EMPIRICAL MOMENTS

STATIST REW03112 05/26/88 09:27:47

VARIABLE	MEAN	STD	FSD	SCEWNESS	KURTOSIS
Edep p,pi+,pi-	1.540E+03	5.538E+02	35.92	1.457E-01	1.846E-01
Esub n	4.548E+02	1.686E+02	37.21	2.205E-01	6.329E-02
Esub pi0	8.984E+02	1.032E+03	114.90	1.461E+00	1.669E+00
Evis p,pi+,pi-	1.214E+02	4.575E+01	37.69	2.040E-01	3.898E-01
Evis n	4.402E+01	1.766E+01	40.71	5.751E-02	2.595E-02
Evis pi0	4.179E+01	5.003E+01	119.72	1.489E+00	1.598E+00
Evis p,pi+,pi-,n,pi0	2.070E+02	4.252E+01	20.54	3.960E-01	4.850E-01

GROUPED MOMENTS

STATIST REW03112 05/26/88 09:27:47

VARIABLE	MEAN	STD	FSD	MEDIAN	MODE
Edep p,pi+,pi-	1.540E+03	5.572E+02	36.18	1.524E+03	1.250E+03
Esub n	4.548E+02	1.657E+02	36.44	4.608E+02	6.660E+02
Esub pi0	1.215E+03	1.032E+03	84.94	8.650E+02	3.500E+02
Evis p,pi+,pi-	1.214E+02	4.585E+01	37.77	1.186E+02	1.050E+02
Evis n	4.402E+01	1.766E+01	40.12	4.390E+01	4.900E+01
Evis pi0	5.657E+01	5.076E+01	89.73	3.971E+01	1.500E+01
Evis p,pi+,pi-,n,pi0	2.070E+02	4.243E+01	20.50	2.091E+02	2.050E+02

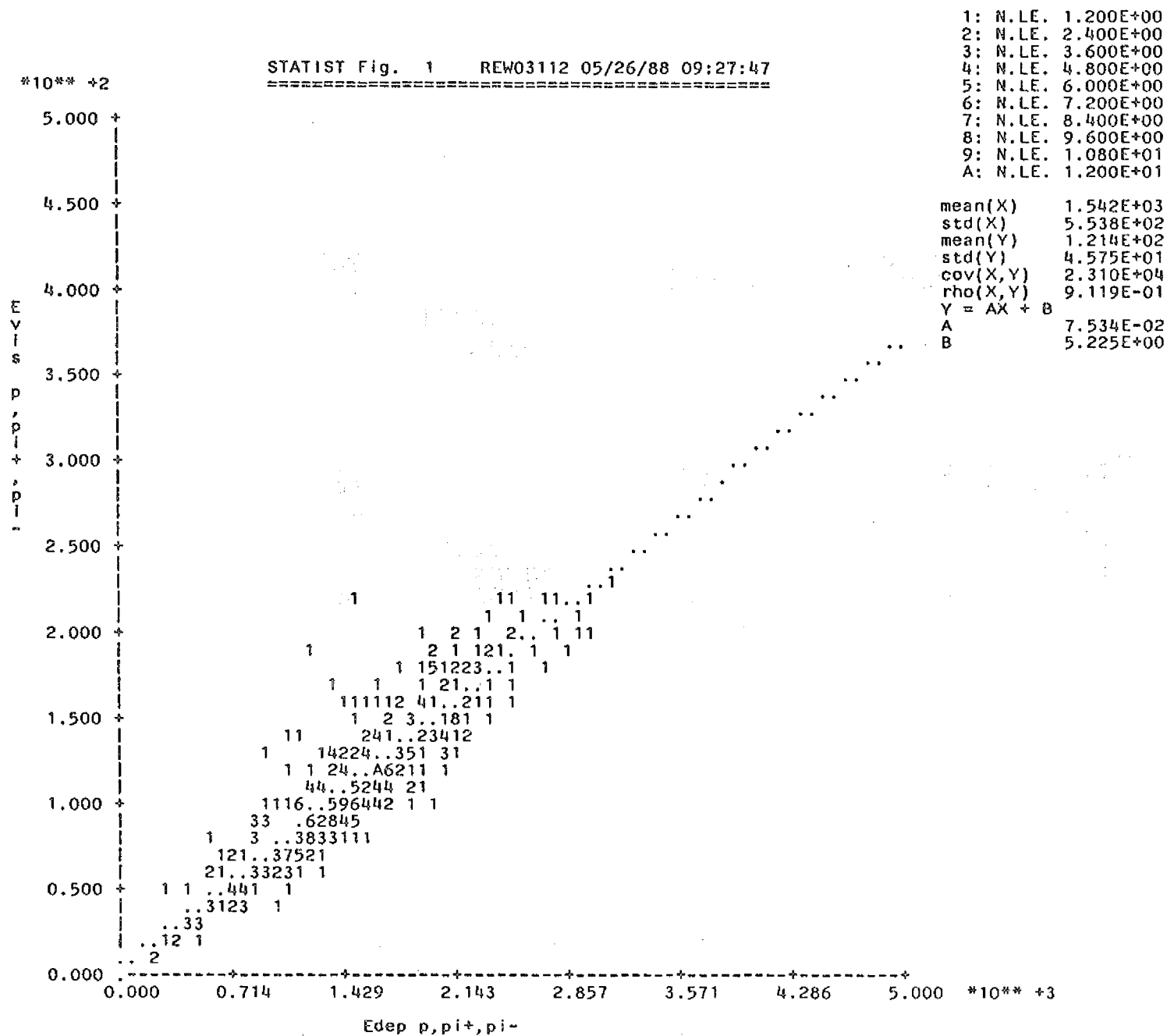
FITTED MOMENTS

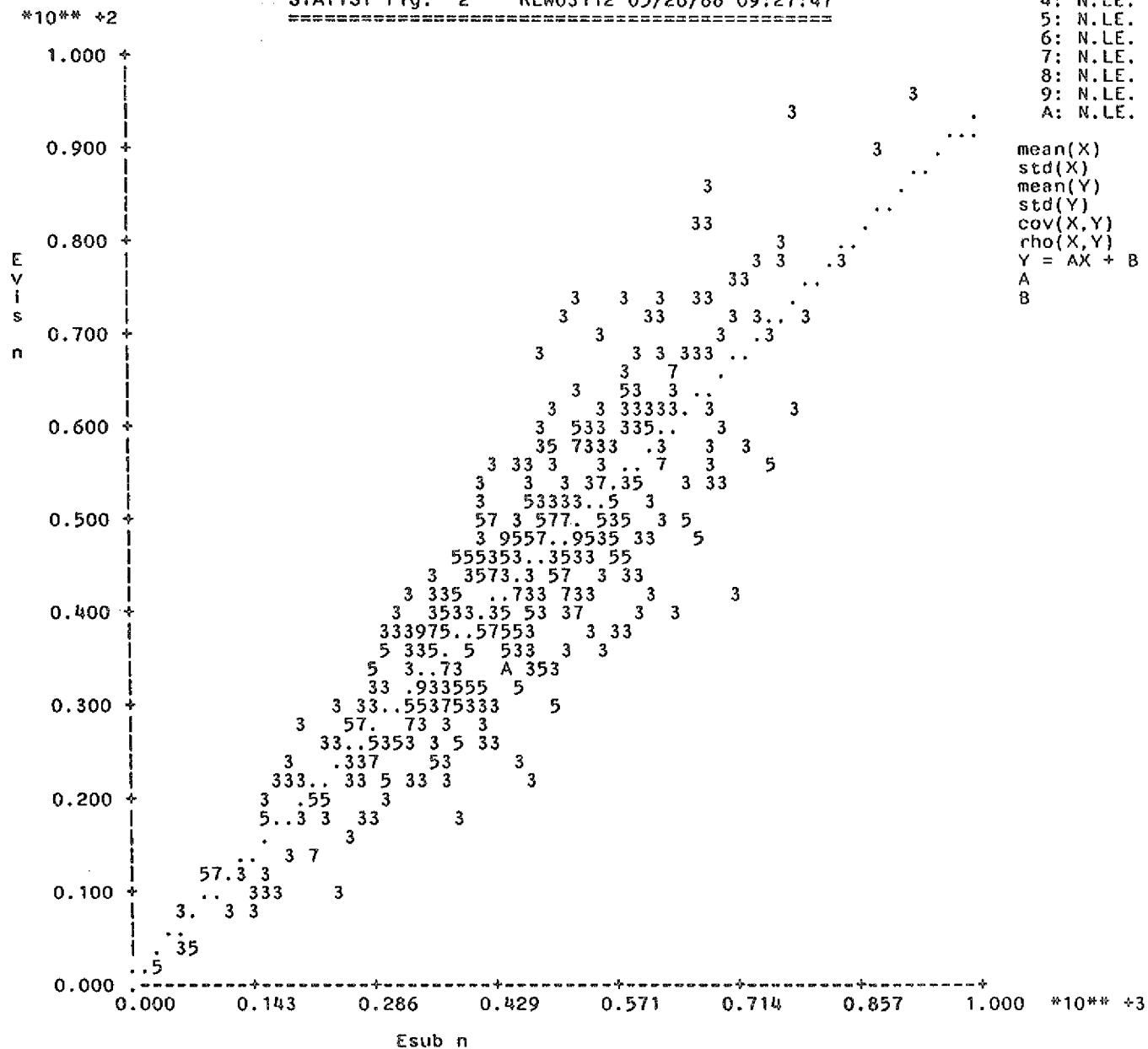
STATIST REW03112 05/26/88 09:27:47

VARIABLE	FROM	TO	N-FIT	MEAN	FSD
Edep p,pi+,pi-	0.000E+00	0.000E+00	0.	0.000E+00	0.00
Esub n	0.000E+00	0.000E+00	0.	0.000E+00	0.00
Esub pi0	0.000E+00	0.000E+00	0.	0.000E+00	0.00
Evis p,pi+,pi-	0.000E+00	0.000E+00	0.	0.000E+00	0.00
Evis n	0.000E+00	0.000E+00	0.	0.000E+00	0.00
Evis pi0	0.000E+00	0.000E+00	0.	0.000E+00	0.00
Evis p,pi+,pi-,n,pi0	1.219E+02	2.920E+02	428.	2.097E+02	17.08

M:STAT3: NUMBER OF EVENTS READ 450

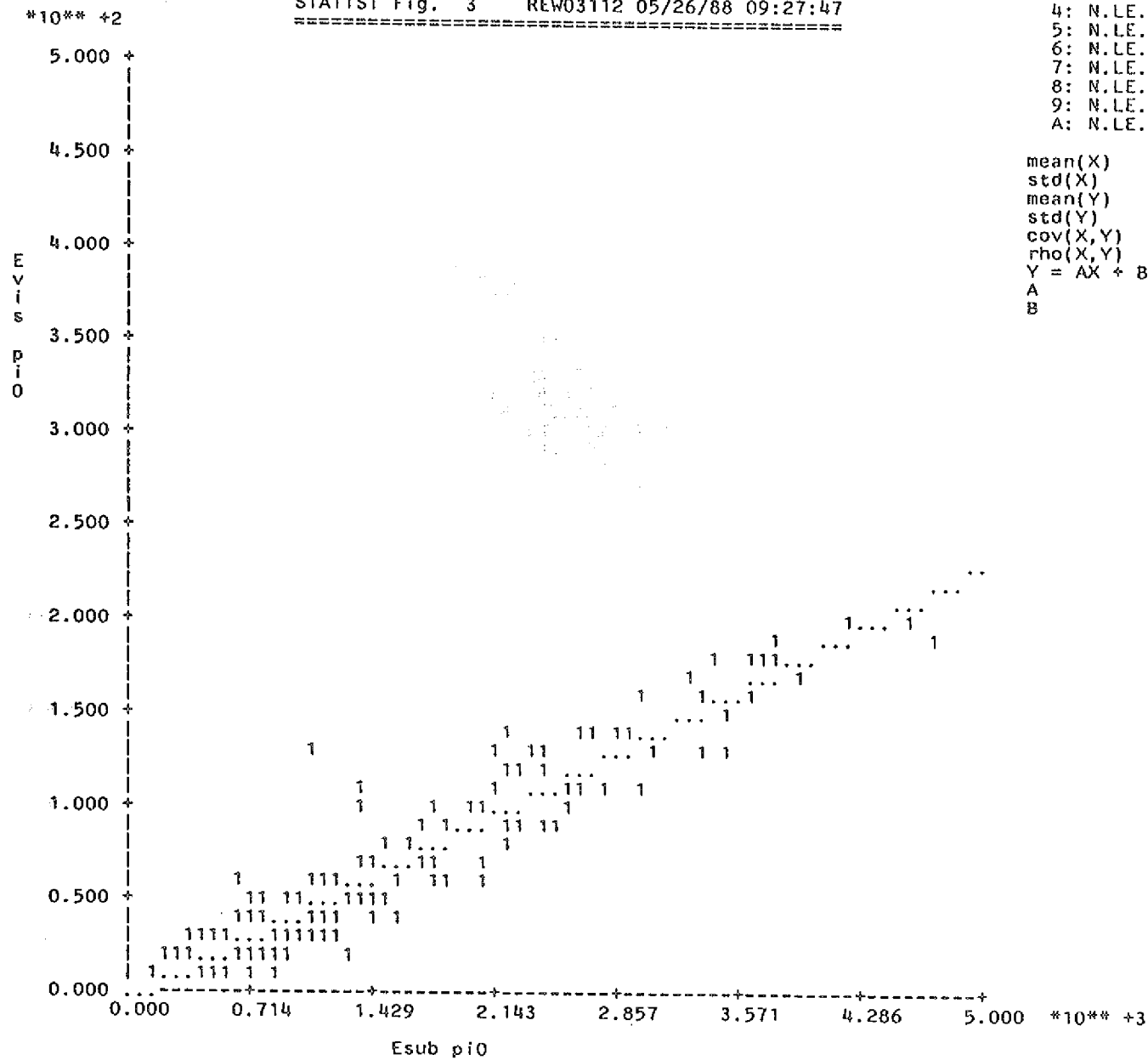
I: EXIT;





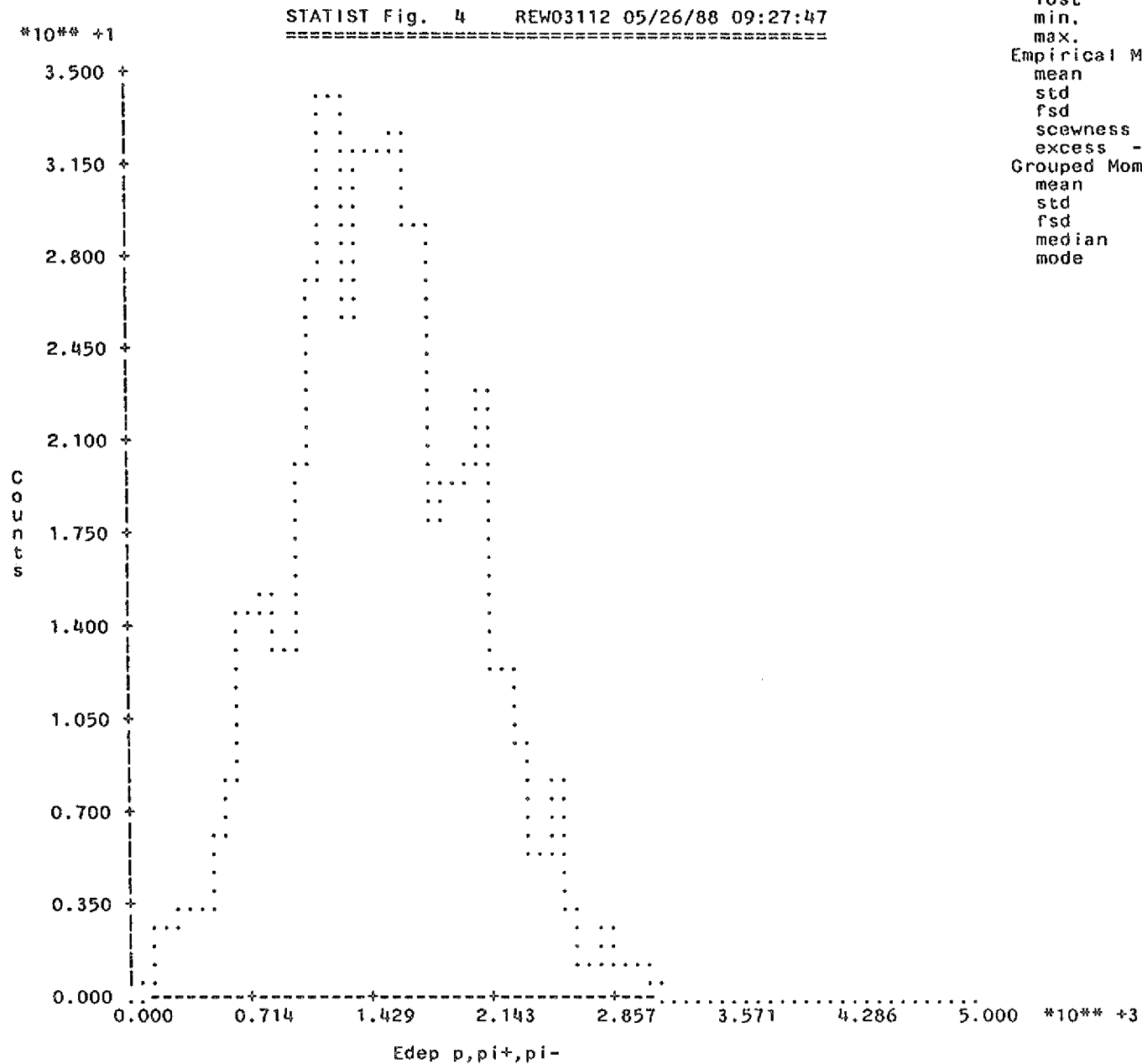
1: N.LE. 5.000E-01
 2: N.LE. 1.000E+00
 3: N.LE. 1.500E+00
 4: N.LE. 2.000E+00
 5: N.LE. 2.500E+00
 6: N.LE. 3.000E+00
 7: N.LE. 3.500E+00
 8: N.LE. 4.000E+00
 9: N.LE. 4.500E+00
 A: N.LE. 5.000E+00

mean(X) 4.530E+02
 std(X) 1.705E+02
 mean(Y) 4.384E+01
 std(Y) 1.797E+01
 cov(X,Y) 2.764E+03
 rho(X,Y) 9.017E-01
 Y = AX + B
 A 9.503E-02
 B 7.834E-01

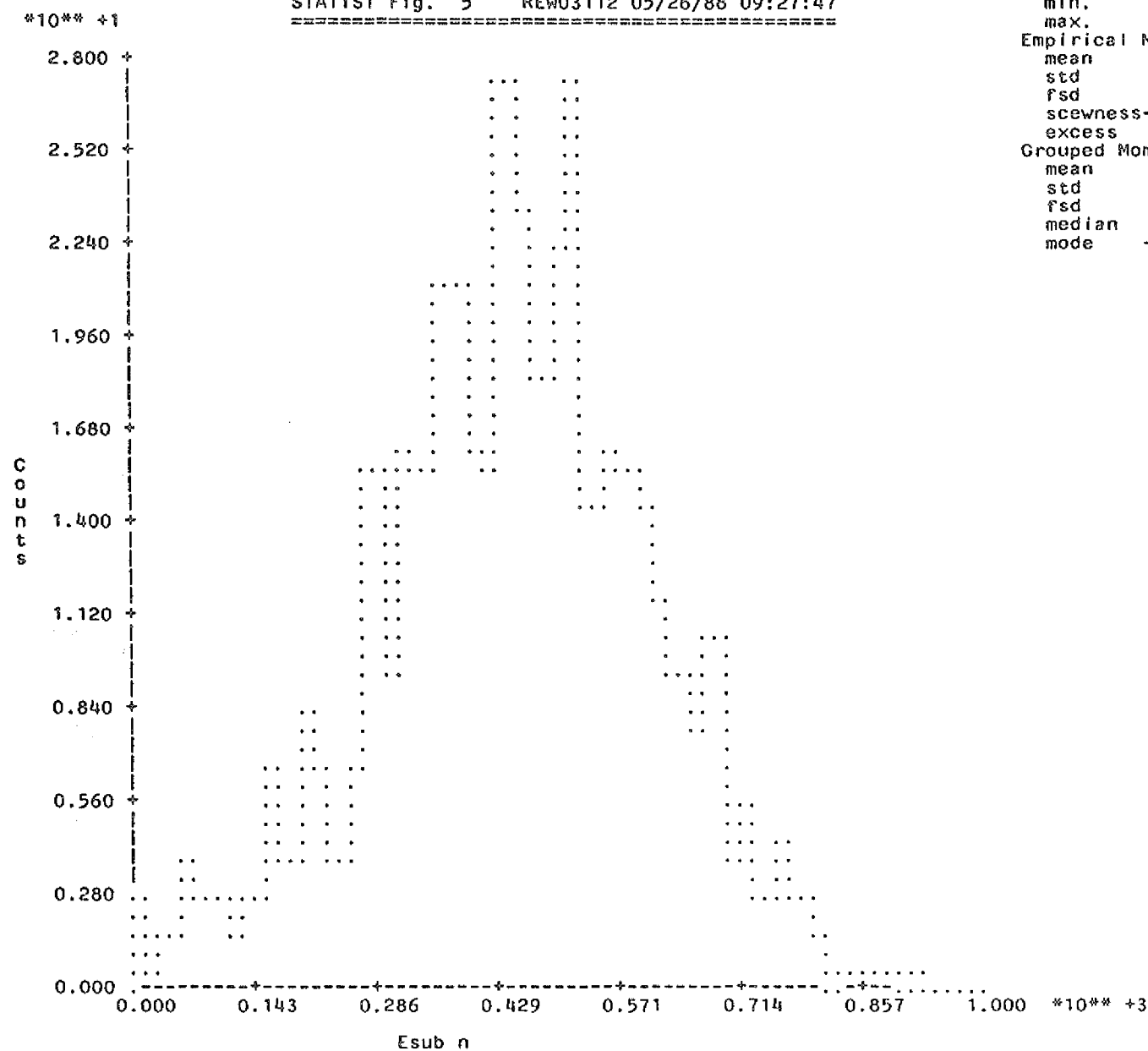


1: N.LE. 1.170E+01
2: N.LE. 2.340E+01
3: N.LE. 3.510E+01
4: N.LE. 4.680E+01
5: N.LE. 5.850E+01
6: N.LE. 7.020E+01
7: N.LE. 8.190E+01
8: N.LE. 9.360E+01
9: N.LE. 1.053E+02
A: N.LE. 1.170E+02

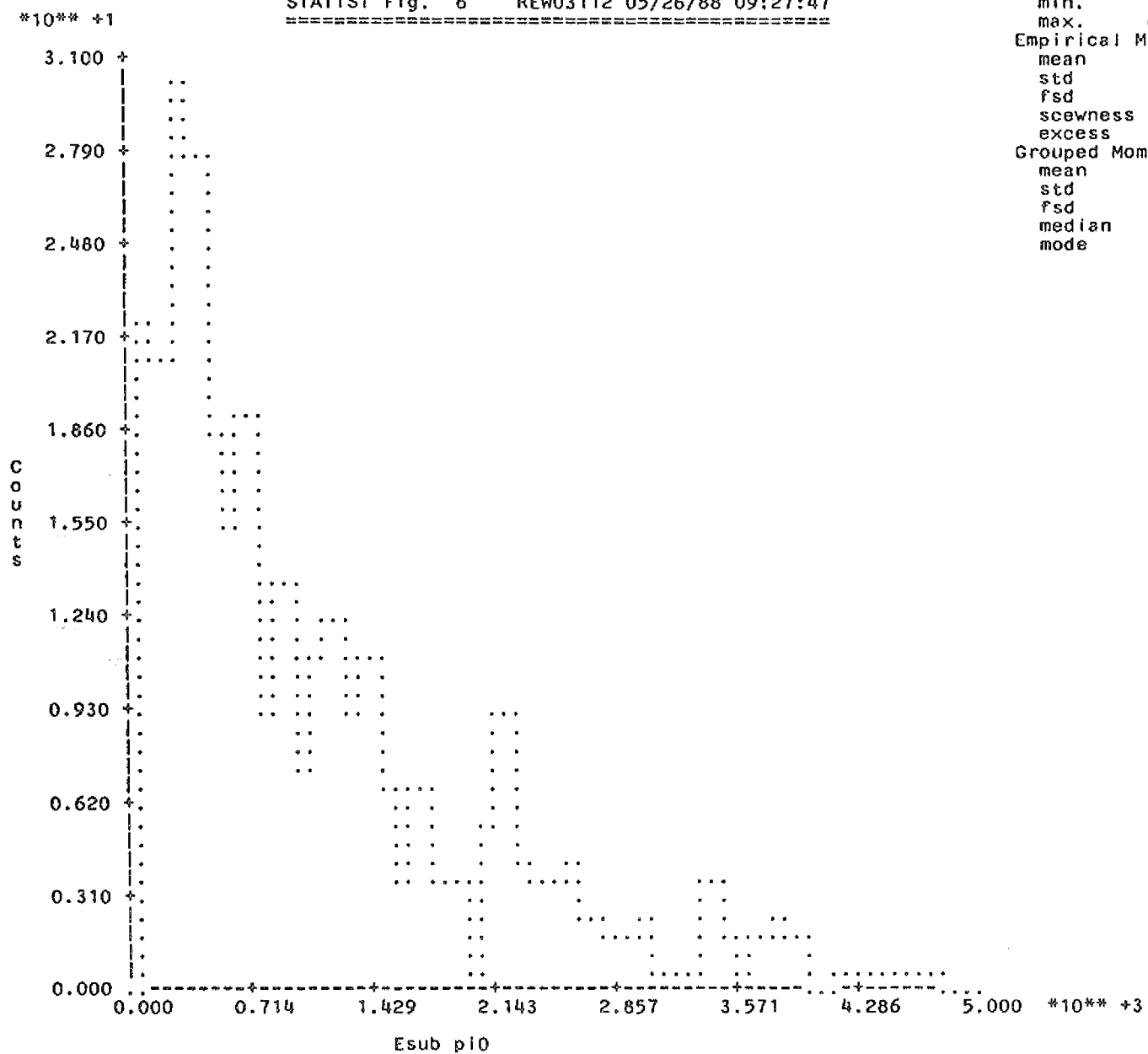
mean(X) 8.984E+02
std(X) 1.032E+03
mean(Y) 4.179E+01
std(Y) 5.004E+01
cov(X,Y) 5.041E+04
rho(X,Y) 9.759E-01
Y = AX + B
A 4.730E-02
B -7.079E-01



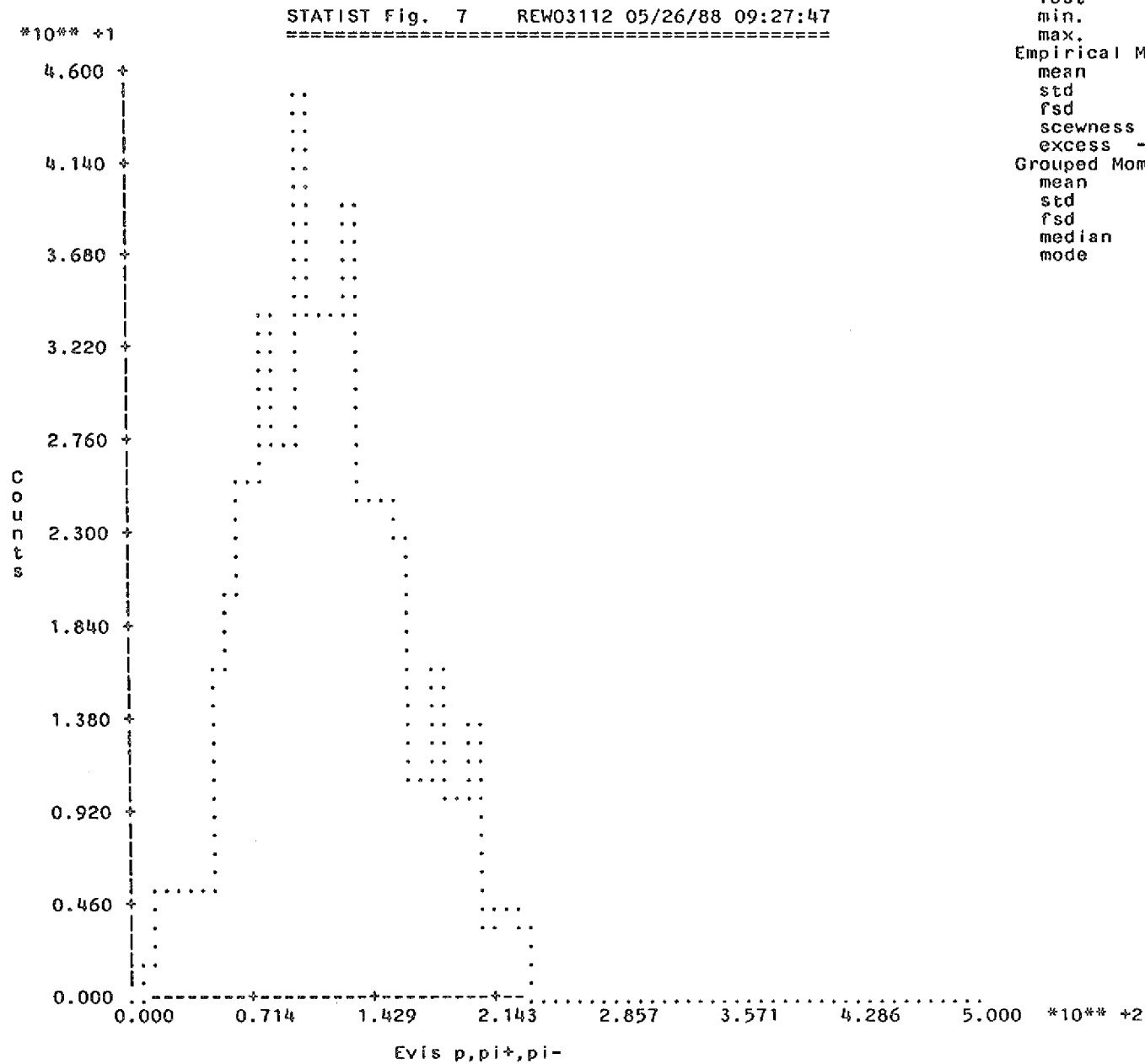
Events	450
0-events	0
lost	0
min.	1.949E+02
max.	3.154E+03
Empirical Moments	
mean	1.542E+03
std	5.538E+02
fsd	35.92%
scwness	1.457E-01
excess	-1.846E-01
Grouped Moments	
mean	1.540E+03
std	5.572E+02
fsd	36.18%
median	1.524E+03
mode	1.250E+03



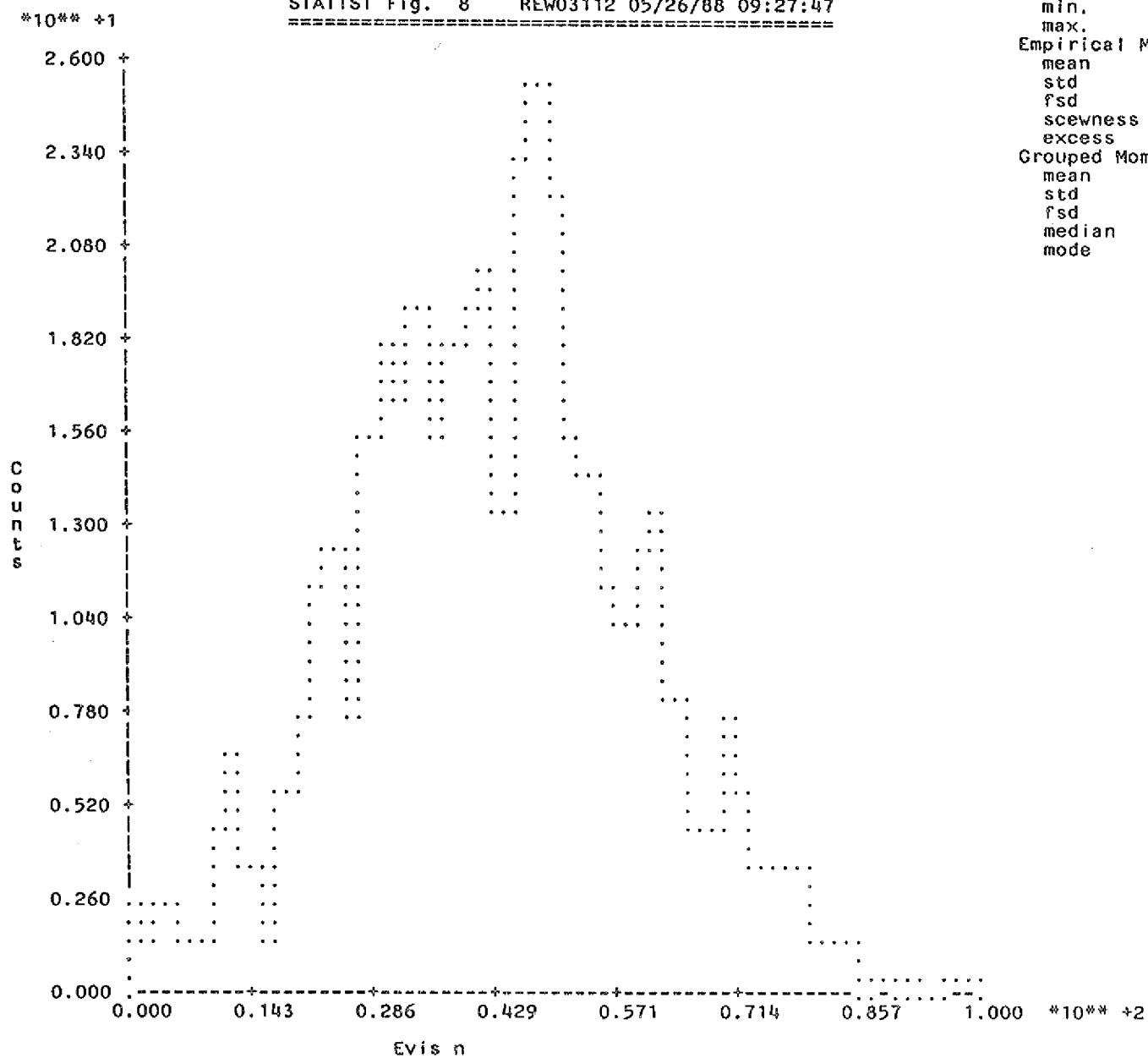
Events	450
0-events	2
lost	0
min.	9.265E+00
max.	9.338E+02
Empirical Moments	
mean	4.530E+02
std	1.686E+02
fsd	37.21%
scwness	-2.205E-01
excess	6.329E-02
Grouped Moments	
mean	4.548E+02
std	1.657E+02
fsd	36.44%
median	4.608E+02
mode	-6.660E+02



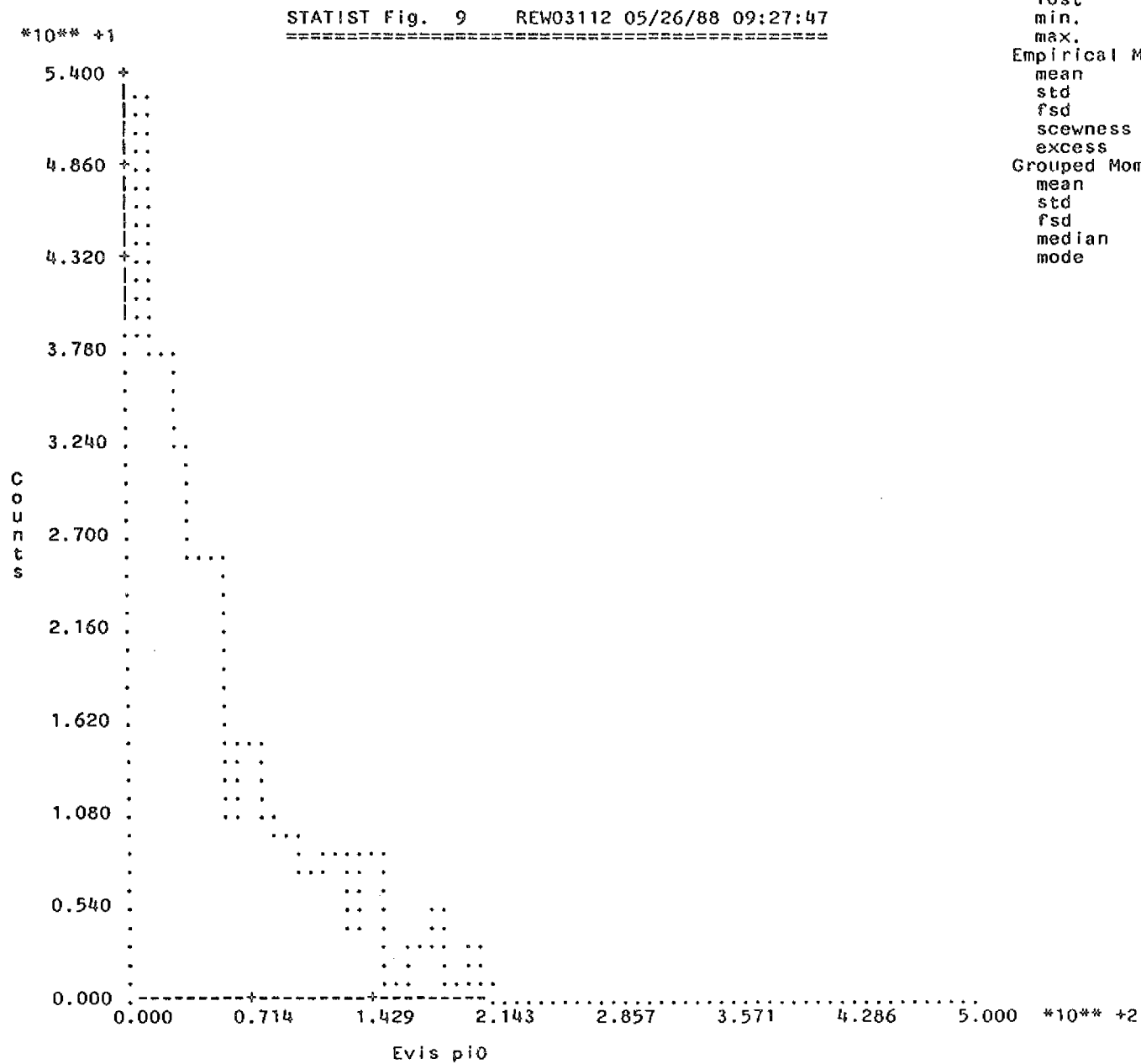
Events	450
0-events	117
lost	0
min.	1.394E+02
max.	4.715E+03
Empirical Moments	
mean	8.984E+02
std	1.032E+03
fsd	114.90%
scawness	1.461E+00
excess	1.669E+00
Grouped Moments	
mean	1.215E+03
std	1.032E+03
fsd	84.94%
median	8.650E+02
mode	3.500E+02



Events	450
0-events	0
lost	0
min.	1.291E+01
max.	2.382E+02
Empirical Moments	
mean	1.214E+02
std	4.575E+01
fsd	37.69%
scwness	2.040E-01
excess	-3.898E-01
Grouped Moments	
mean	1.214E+02
std	4.585E+01
fsd	37.77%
median	1.186E+02
mode	1.050E+02



Events	450
0-events	2
lost	0
min.	3.501E-01
max.	9.972E+01
Empirical Moments	
mean	4.384E+01
std	1.785E+01
fsd	40.71%
scwness	5.751E-02
excess	2.595E-02
Grouped Moments	
mean	4.402E+01
std	1.766E+01
fsd	40.12%
median	4.390E+01
mode	4.900E+01



Events	450
O-events	118
lost	0
min.	1.809E+00
max.	2.120E+02
Empirical Moments	
mean	4.179E+01
std	5.003E+01
fsd	119.72%
scwness	1.489E+00
excess	1.598E+00
Grouped Moments	
mean	5.657E+01
std	5.076E+01
fsd	89.73%
median	3.971E+01
mode	1.500E+01

*10** +1

4.855 +
4.369 +
3.884 +
3.398 +
2.913 +
2.427 +
1.942 +
1.456 +
0.971 +
0.485 +
0.000

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STATIST Fig. 10 REW03112 05/26/88 09:27:47
=====

Evis p,pi+,pi-,n,pi0

0.000 0.714 1.429 2.143 2.857 3.571 4.286 5.000 *10** +2

Events 450
0-events 0
lost 0
min. 5.315E+01
max. 3.310E+02
Empirical Moments
mean 2.070E+02
std 4.252E+01
fsd 20.54%
scewness-3.960E-01
excess 4.850E-01
Grouped Moments
mean 2.070E+02
std 4.243E+01
fsd 20.50%
median 2.091E+02
mode 2.050E+02
Gauss - Fit
from 1.219E+02
to 2.920E+02
events 428
mean 2.097E+02
std 3.581E+01
fsd 17.08%

E.2 Sample Problem II

Double-Differential Cross-Section Calculation

This is an example using the "thin"-target option of HETC to calculate double-differential cross-sections. This test case considers an 800 MeV proton beam on lead.

E.2.1 Input Files for Sample Problem II

(1) Input to HETC and SIM

```
//REW101PB JOB ($$ACCOUNT$),THIN,TIME=(60),REGION=3000K,
//  CLASS=K,MSGCLASS=A
/*JOBPARM  L=15
/* PB-208  800.MEV P  ON PB THIN TARGET, PROTON  YIELDS
/*PASSWORD $MVS
/*ROUTE  PRINT CMS
//HET86  EXEC  LGFORTV,COND=(9,LT)
//L.SYSPRINT DD DUMMY
//L.SYSUT1 DD UNIT=TVIO,SPACE=(6400,180)
//L.LOD31  DD DSN=REW031.HERMES,DISP=SHR
//L.SYSIN DD *
      INCLUDE LOD31(THIN)
      INCLUDE LOD31(HETKFA2)
      ENTRY HET
/*
//G.FT20F001 DD DSN=REW071.HET86.BERT,DISP=SHR,LABEL=(,,IN)      BERTP
//G.FT26F001 DD SYSOUT=*,DCB=(RECFM=UA,BLKSIZE=133)             ECHO
//G.SYSIN DD *
      REAL THE(5);DATA THE=< 0. 15. 45. 90. 180. >
      REAL E(33);DATA E=< 800. A 14 -50. A 3 -25. A 4 -5. A 5 -1. >
      DEFDET CASP,1,0.,$Z,E,THE;
      DEFDET EVAP,2,0.,$Z,E,THE;
      DEFDET YLDP,3,0.,$Z,E,THE;
      DEFDET CP00,1,0.;
      DEFDET CN00,1,1.;
RUN;
&RUN
      MAXCAS=500,  MAXBCH=20,  IRUN=1,  ISEED=123456789,
      EMAX=800.,  ESR=800.,  WSR=1.,  TIPSR=0.,

      EMIN(1)=1.,  EMIN(2)=15.,  EPICUT=2.23,  EMUCUT=1.69,  ELOP=20.,
      IOSUB=0 ,  IBERTP=-20,  NHSTP=0,
      IELAS=0,  ISTRAG=0,  IANG=1,  IFISS=0,KEYDK=0,  IB0=1,  B0=8.,

      MXMAT=1,
      NEL(1)=1,  DENH(1)=0.,
      ZZ(1,1)=82.,A(1,1)=208.,DEN(1,1)= 1.000,

&END
EXIT;
/*
```

E.2.2 Output File Excerpt for Sample Problem II

(1) Output of HETC and SIM

```

/JOB-ID = REW101PB 05/02/88 23:53:05
I: REAL THE(5);DATA THE=< 0. 15. 45. 90. 180. >
I: REAL E(33);DATA E=< 800. A 14 -50. A 3 -25. A 4 -5. A 5 -1. >
I: DEFDET CASP,1,0.,$Z,E,THE;
I: DEFDET EVAP,2,0.,$Z,E,THE;
I: DEFDET YLDP,3,0.,$Z,E,THE;
I: DEFDET CPOO,1,0.;
I: DEFDET CNOO,1,1.;
I:RUN;
MAXBCH= 20 IBERTP=-20 EMIN(1)= 1.000E+00 ELOP = 2.000E+01
MAXCAS= 500 IOSUB = 0 EMIN(2)= 1.500E+01 CTOFE = 0.000E+00
IRUN = 1 NHSTP = 0 EPICUT = 2.230E+00 EMAX = 8.000E+02
EMUCUT = 1.690E+00 BO = 8.000E+00
NICOL =0 NPIDK =1 NBOGUS =1 NSEUDO=1
NSPRD=0 NWSPRD=0 ISTRAG =0 IANG =1
IFISS =0 IBO =1 IGEOM =0 KEYDK =0
IELAS =0
ISEED = 123456789
LHI =0 HIA(5)= 0.0 HIZ(5)= 0.0 HIBE(5)= 0.000E+00
/BERTTP(I): I-N-C DATA READ FORM UNIT 20
/BERTTP(I): EVAP DATA READ FORM UNIT 20
NON-ISOTROPIC ANGLE OPTION SELECTED
HSIGMX 0.12000E-23 0.64500E-24 0.20000E-24 0.00000E+00 0.67700E-25
THIS IS MEDIUM 1
DENH=0.000000E+00 NEL= 1 SO= 1.000E+00 KB= 0.000E+00
ZZ( 1, 1)=82.0 A( 1, 1)=208.0 DEN( 1, 1)=1.000000E+00
"GEOMETRIC" X-SECTIONS IN (1/CM)

  NUCLIDS:      1      2      3      4      5      6
7      8      9     10
  MEDIA
  1      2.4615E+00
MAXIMUM H-1 X-SECTIONS IN (1/CM)

  MEDIA      PROTONS      NEUTRONS      PI+      PIO      PI-
  1      0.0000E+00 0.0000E+00 0.0000E+00 0.0000E+00 0.0000E+00
TRANSPORT X-SECTION = SUM("GEOMETRIC") + MAX. H-1 + MAX. DECAY X-SECTIONS IN (1/CM)

  MEDIA      PROTONS      NEUTRONS      PI+      PIO      PI-      MUE+
MUE-
  1      2.4615E+00 2.4615E+00 2.4686E+00 0.0000E+00 2.4686E+00 8.8943E-05
8.8943E-05
SOURCE DATA:
TYP = 0.
XYZ = 0.000 0.000 0.000 GAUSSIAN DISTRIBUTED WITH
XYZ SIG.= 0.000 0.000 0.000 TRUNCATED TO RADIUS
R = 0.000 IF R>0
UVW = 0.0000 0.0000 1.0000 IF = 0.,0.,0. -> ISOTROPIC
WT = 1.000
E = 800.000 GAUSSIAN DISTRIBUTED WITH
E SIG.= 0.000
M: YOU ARE USING THIN TARGET SETUP
--> SOR HET IRUN= 1 ISEED= 123456789
--> SOB HET IBAT= 1 ISEED= 123456789
/BERTTP(I): I-N-C DATA READ FORM UNIT 20
/DRES(I): USING 1977 WAPS DATA
--> EOB HET IBAT= 1 ISEED= 876852125 NHIS= 500 CPU= 45.57SEC.
  MEDIUM  SORS  CASC  SLOW  ESCP  PSEU  ABSO  BRDY  SCAT
      1    500   361    0    0   139    0    0    0    0
    6666    0    0    0    0    0    0    0    0    0
  TOTAL   500   361    0    0   139    0    0    0    0

```

```
--> SOB HET IBAT= 2 ISEED= 876852125
--> EOB HET IBAT= 2 ISEED= 1939235471 NHIS= 1000 CPU= 40.19SEC.
MEDIUM SORS CASC SLOW ESCP PSEU ABSO BRDY SCAT
```

```
1 500 356 0 0 144 0 0 0 0
6666 0 0 0 0 0 0 0 0 0
TOTAL 500 356 0 0 144 0 0 0 0
```

```
--> EOR HET IRUN= 1 ISEED= 1734564103 NHIS=10000 CPU= 782.97SEC.
MEDIUM SORS CASC SLOW ESCP PSEU ABSO BRDY SCAT
```

```
1 10000 7155 0 0 2845 0 0 0 0
6666 0 0 0 0 0 0 0 0 0
TOTAL 10000 7155 0 0 2845 0 0 0 0
```

```
REAL PSEUDO REAL PSEUDO PSEUDO
REAL PSEUDO
NONHYDROG NONHYDROG HYDROGEN HYDROGEN DECAY
ELASTIC ELASTIC
COLLISIONS COLLISIONS COLLISIONS COLLISIONS COLLISIONS
COLLISIONS COLLISIONS
MEDIUM 1 0.71550E+04 0.28450E+04 0.00000E+00 0.00000E+00 0.00000E+00
0.00000E+00 0.00000E+00
```

PSEUDO COLLISIONS WITHIN INTERNAL VOID BY

PARTICLES OF

```
TYPE 1 TYPE 2 TYPE 3 TYPE 4 TYPE 5
TYPE 6 TYPE 7
0.00000E+00 0.00000E+00 0.00000E+00 0.00000E+00 0.00000E+00
0.00000E+00 0.00000E+00
--> NEGEX= 0 LOWAZ= 0 NWTC= 0
MEDIUM SOURCE-FRACTION %ERR
1 1.000E+00 0.00
VOID 0.000E+00 0.00
MEDIUM INELASTIC C./SOURCE %ERR ELASTIC C./SOURCE %ERR
1 7.155E-01 0.53 0.000E+00 0.00
VOID 0.000E+00 0.00 0.000E+00 0.00
MEDIUM ABSORPTION /SOURCE %ERR FISSION /SOURCE %ERR
1 0.000E+00 0.00 0.000E+00 0.00
VOID 0.000E+00 0.00 0.000E+00 0.00
MEDIUM P FLUX (CM) %ERR N FLUX (CM) %ERR
1 0.000E+00 0.00 0.000E+00 0.00
VOID 0.000E+00 0.00 0.000E+00 0.00
MEDIUM PI+ FLUX (CM) %ERR PI- FLUX (CM) %ERR
1 0.000E+00 0.00 0.000E+00 0.00
VOID 0.000E+00 0.00 0.000E+00 0.00
MEDIUM MU+ FLUX (CM) %ERR MU- FLUX (CM) %ERR
1 0.000E+00 0.00 0.000E+00 0.00
VOID 0.000E+00 0.00 0.000E+00 0.00
MEDIUM P YIELD %ERR N YIELD %ERR
1 1.685E+00 0.94 9.687E+00 0.72
VOID 0.000E+00 0.00 0.000E+00 0.00
MEDIUM PI+ YIELD %ERR PIO YIELD %ERR
1 9.640E-02 1.59 1.097E-01 2.74
VOID 0.000E+00 0.00 0.000E+00 0.00
MEDIUM PI- YIELD %ERR
1 6.220E-02 4.19
```

VOID		0.000E+00	0.00			
MEDIUM	MU+	YIELD	%ERR	MU-	YIELD	%ERR
1		0.000E+00	0.00		0.000E+00	0.00
VOID		0.000E+00	0.00		0.000E+00	0.00
MEDIUM	D	YIELD	%ERR	T	YIELD	%ERR
1		5.790E-01	1.28		5.444E-01	1.79
VOID		0.000E+00	0.00		0.000E+00	0.00
MEDIUM	HE3	YIELD	%ERR	HE4	YIELD	%ERR
1		2.680E-02	6.84		1.130E+00	1.12
VOID		0.000E+00	0.00		0.000E+00	0.00

W:RSYS: NO RSYST-TAPE AVAILABLE
 YIELD DETECTOR FOR P FROM INC
 DETECTOR NAME = CASP
 THETA1= 0.000E+00 THETA2= 1.500E+01

E-HIGH	E-LOW	INC-YIELD	%ERR
8.000E+02	7.500E+02	1.930E-02	6.48
7.500E+02	7.000E+02	1.050E-02	11.38
7.000E+02	6.500E+02	1.200E-03	18.73
6.500E+02	6.000E+02	1.200E-03	35.04
6.000E+02	5.500E+02	2.100E-03	17.58
5.500E+02	5.000E+02	3.500E-03	19.39
5.000E+02	4.500E+02	2.800E-03	22.23
4.500E+02	4.000E+02	5.100E-03	14.36
4.000E+02	3.500E+02	5.800E-03	13.21
3.500E+02	3.000E+02	4.900E-03	15.24
3.000E+02	2.500E+02	5.200E-03	12.60
2.500E+02	2.000E+02	6.300E-03	13.08
2.000E+02	1.500E+02	5.300E-03	12.63
1.500E+02	1.000E+02	6.300E-03	10.87
1.000E+02	7.500E+01	3.000E-03	19.04
7.500E+01	5.000E+01	5.100E-03	14.91
5.000E+01	2.500E+01	7.300E-03	9.38
2.500E+01	2.000E+01	1.900E-03	24.72
2.000E+01	1.500E+01	4.200E-03	20.11
1.500E+01	1.000E+01	3.300E-03	17.18
1.000E+01	5.000E+00	6.000E-04	42.58
5.000E+00	4.000E+00	0.000E+00	0.00
4.000E+00	3.000E+00	0.000E+00	0.00
3.000E+00	2.000E+00	0.000E+00	0.00
2.000E+00	1.000E+00	0.000E+00	0.00
1.000E+00	0.000E+00	0.000E+00	0.00
0.000E+00	0.000E+00	0.000E+00	0.00
0.000E+00	0.000E+00	0.000E+00	0.00
0.000E+00	0.000E+00	0.000E+00	0.00
0.000E+00	0.000E+00	0.000E+00	0.00
0.000E+00	0.000E+00	0.000E+00	0.00
0.000E+00	0.000E+00	0.000E+00	0.00
0.000E+00	0.000E+00	0.000E+00	0.00

THETA1= 1.500E+01 THETA2= 4.500E+01

E-HIGH	E-LOW	INC-YIELD	%ERR
8.000E+02	7.500E+02	9.000E-04	34.10
7.500E+02	7.000E+02	7.800E-03	13.01
7.000E+02	6.500E+02	1.340E-02	8.60
6.500E+02	6.000E+02	8.300E-03	10.08
6.000E+02	5.500E+02	8.300E-03	8.06
5.500E+02	5.000E+02	9.500E-03	6.81
5.000E+02	4.500E+02	1.060E-02	10.16
4.500E+02	4.000E+02	1.340E-02	7.43
4.000E+02	3.500E+02	1.570E-02	7.17
3.500E+02	3.000E+02	1.760E-02	6.57
3.000E+02	2.500E+02	2.420E-02	7.70
2.500E+02	2.000E+02	2.590E-02	7.01
2.000E+02	1.500E+02	3.610E-02	5.17
1.500E+02	1.000E+02	3.900E-02	4.28

1.000E+02	7.500E+01	2.730E-02	5.83
7.500E+01	5.000E+01	3.190E-02	4.95
5.000E+01	2.500E+01	4.670E-02	4.95
2.500E+01	2.000E+01	1.540E-02	7.94
2.000E+01	1.500E+01	2.640E-02	5.67
1.500E+01	1.000E+01	2.620E-02	7.30
1.000E+01	5.000E+00	6.400E-03	12.54
5.000E+00	4.000E+00	0.000E+00	0.00
4.000E+00	3.000E+00	0.000E+00	0.00
3.000E+00	2.000E+00	0.000E+00	0.00
2.000E+00	1.000E+00	0.000E+00	0.00
1.000E+00	0.000E+00	0.000E+00	0.00
0.000E+00	0.000E+00	0.000E+00	0.00
0.000E+00	0.000E+00	0.000E+00	0.00
0.000E+00	0.000E+00	0.000E+00	0.00
0.000E+00	0.000E+00	0.000E+00	0.00
0.000E+00	0.000E+00	0.000E+00	0.00
0.000E+00	0.000E+00	0.000E+00	0.00
0.000E+00	0.000E+00	0.000E+00	0.00
0.000E+00	0.000E+00	0.000E+00	0.00
0.000E+00	0.000E+00	0.000E+00	0.00

YIELD DETECTOR FOR P FROM INC

DETECTOR NAME = CP00

INC-YIELD	%ERR
1.401E+00	0.92

YIELD DETECTOR FOR N FROM INC

DETECTOR NAME = CN00

INC-YIELD	%ERR
2.800E+00	0.75

I:EXIT;

MESSAGE SUMMARY: MESSAGE NUMBER - COUNT

212	9
-----	---

E.3 Sample Problem III

Thick-Target Particle Flux Calculation

This is an example for particle flux calculation performed on a "thick" target formed by an iron cylinder as depicted in Fig. E.3. The fluxes are determined in different regions, energy bins and for different particle types. The target was hit axially by a 1 GeV proton pencil beam.

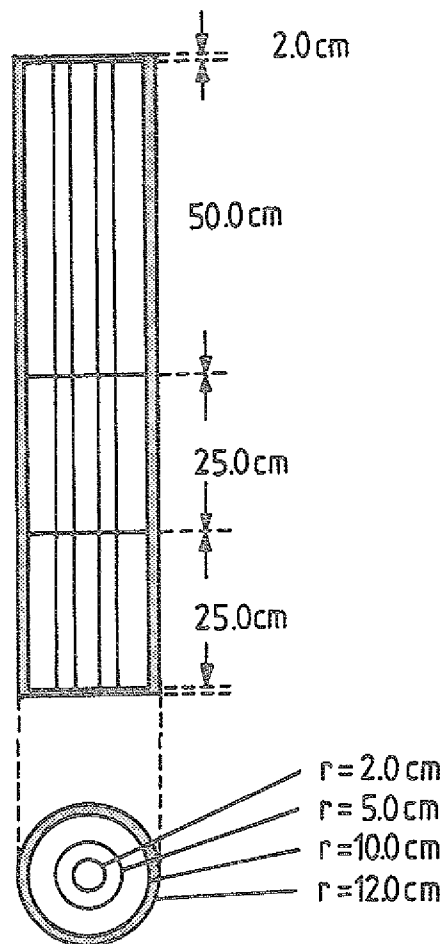


Fig. E.3: Geometrical Setup

E.3.1 Input Files for Sample Problem III

(1) Geometry Description in CMD Language

```

/ Test3 geometry
GEOCMP 8 13 13 ;
/ Body input
PZ 1 0.;
PZ 2 25.;
PZ 3 50.;
PZ 4 100.;
CZ 11 $N 2.;
CZ 12 $N 5.;
CZ 13 $N 10.;
RCC 20 0. 0. -1. 0. 0. 102. 12.;
END;
/ Zone input
ZONE 1 11 +11 +1 -2;
ZONE 1 12 +11 +2 -3;
ZONE 1 13 +11 +3 -4;
ZONE 1 21 +12 -11 +1 -2;
ZONE 1 22 +12 -11 +2 -3;
ZONE 1 23 +12 -11 +3 -4;
ZONE 1 31 +13 -12 +1 -2;
ZONE 1 32 +13 -12 +2 -3;
ZONE 1 33 +13 -12 +3 -4;
ZONE 6666 61 +20 +13 -1;
ZONE 6666 62 +20 +13 +4;
ZONE 6666 63 +20 -13;
ZONE -1 90 -20;
ENDGEO;
/ Load geometry object file from unit 22;
GEOIN;
/ Return to terminal input for test
UNIT 5;

```


(2) Input to HETC and SIM

```

//REW031H3 JOB ($$ACCOUNT$),'TEST3-HET',TIME=(4),REGION=3000K,
//  CLASS=C,MSGCLASS=A
/*JOBPARM L=15
/*PASSWORD $MVS
/*ROUTE PRINT CMS
//HET EXEC GFORTV,NAME=HETKFA2,DS='REW031.HERMES'
//G.FT20F001 DD DSN=REW071.HET86.BERT,DISP=SHR,LABEL=(,,IN)      BERTP
//G.FT21F001 DD DSN=REW031.HETSUB.TEST3,DISP=SHR                IOSUB
//G.FT07F001 DD SYSOUT=P,DCB=(RECFM=F,LRECL=80,BLKSIZE=80),
//  DEST=(CMS,REW031)
//G.FT25F001 DD *                                              INGEO
$INCLUDE TEST3 GEO-0
/*
//G.SYSIN DD *
/ setup detector regions
  INTEGER IR1(9);DATA IR1=<11 12 13 21 22 23 31 32 33>;
/ setup detector energy bins
  REAL EP(12),EN(9);
  DATA EP=< 1000. 750. 500. 250. 100. 50. 25. 20. 15. 10. 5. 1.>;
  DATA EN=< 1000. 750. 500. 250. 100. 50. 25. 20. 15. >;
/ setup detectors
  DEFDET PHIP,8,0.,,EP,,IR1;
  DEFDET PHIN,8,1.,,EN,,IR1;
RUN; / return to het primary input
&RUN
  MAXCAS=20,  MAXBCH= 50,
  EMAX=1000., EMIN(1)=1., EMIN(2)=14.9, ELOP=20.,
  IBERTP=-20, IOSUB=21, NHSTP= 0, INGEO=25,
  IELAS=2, IANG=1, IFISS=0,

  MXMAT=1,    DENH=0., NEL=1, ZZ=26., A=56., DEN=.08460,

  TIPSR=0., ESR= 1000. , ZSR=-.5, WSR=1.,
&END
/ PHI.n(E,z,r) is available on PHIN@EOR
/ PHI.p(E,z,r) is available on PHIP@EOR
/ save detector output for later processing with CMD
/ TEST3 DETECTOR OUTPUT
SAVE PHIN@EOR;
SAVE PHINFSD;
SAVE PHIP@EOR;
SAVE PHIPFSD;
EXIT;
/*

```

(3) Input to MORSE

```

//REW101$$ JOB ($$ACCOUNT$),TEST3,TIME=(128),REGION=6000K,CLASS=C
/*PASSWORD $MVS
/*JOBPARM L=20
/*ROUTE PRINT CMS
// EXEC LGFORTV
//L.LOAD DD DSN=REW031.HERMES,DISP=SHR
//L.SYSIN DD *
    INCLUDE LOAD(MORSE)
    INCLUDE LOAD(CGL2)
    ENTRY MORSCG
//G.FT50F001 DD DSN=REW031.HETSUB.TEST3,DISP=SHR,LABEL=(,,IN)
//G.FT20F001 DD DSN=REW031.EPRLIB,DISP=SHR,LABEL=(,,IN)
//*.FT21F001 DD DSN=REW101.TEST3.MORDET,DISP=SHR
//G.SYSIN DD *
    HERMES TEST3      9 DETEKTOREN      CARD A
      0 0 32 6 / CARD A2
    200 2500 50 1 100 21 100 121 0 0 58 1 0/ CARD B
      1 1 0 0 1.0E+0 1.0E-4 1.0E+4 1.0E-7 .6293E+6 / CARD C
      0. 0. 0. 0. 0. 0. 0. / CARD D
      50 / CARD D2
      / ENERGIES (EV) CARD F
    0.149E+08 0.135E+08 0.122E+08 0.111E+08 0.100E+08 0.905E+07 0.819E+07 /
    0.741E+07 0.670E+07 0.607E+07 0.549E+07 0.497E+07 0.449E+07 0.407E+07 /
    0.368E+07 0.333E+07 0.301E+07 0.273E+07 0.247E+07 0.223E+07 0.202E+07 /
    0.183E+07 0.165E+07 0.150E+07 0.135E+07 0.122E+07 0.111E+07 0.100E+07 /
    0.907E+06 0.821E+06 0.743E+06 0.672E+06 0.608E+06 0.550E+06 0.498E+06 /
    0.450E+06 0.408E+06 0.369E+06 0.334E+06 0.302E+06 0.273E+06 0.247E+06 /
    0.224E+06 0.202E+06 0.183E+06 0.166E+06 0.150E+06 0.136E+06 0.123E+06 /
    0.111E+06 0.865E+05 0.674E+05 0.525E+05 0.409E+05 0.318E+05 0.248E+05 /
    0.193E+05 0.150E+05 0.117E+05 0.912E+04 0.710E+04 0.553E+04 0.431E+04 /
    0.335E+04 0.261E+04 0.203E+04 0.158E+04 0.123E+04 0.961E+03 0.749E+03 /
    0.583E+03 0.454E+03 0.354E+03 0.275E+03 0.214E+03 0.167E+03 0.130E+03 /
    0.101E+03 0.789E+02 0.614E+02 0.479E+02 0.373E+02 0.290E+02 0.226E+02 /
    0.176E+02 0.137E+02 0.107E+02 0.832E+01 0.648E+01 0.504E+01 0.393E+01 /
    0.306E+01 0.238E+01 0.186E+01 0.144E+01 0.113E+01 0.876E+00 0.683E+00 /
    0.532E+00 0.414E+00 0.140E+08 0.120E+08 0.100E+08 0.800E+07 0.750E+07 /
    0.700E+07 0.650E+07 0.600E+07 0.550E+07 0.500E+07 0.450E+07 0.400E+07 /
    0.350E+07 0.300E+07 0.250E+07 0.200E+07 0.150E+07 0.100E+07 0.400E+06 /
    0.200E+06 0.100E+06 ; E O CARD F
    000035FA731A / CARD H
      1 1 0 0 0 1 121 0 / CARD I
      0 0 0 0 0 0 1.0E+0 1.00E+0 1.0E+0 0.0E+0 / CARD J
      -1 / E O CARD J
      0 0 0 0 / CARD L
      F 1. ; GAMMA PROB.'S GWLO(NMTG,MXREG) CARD O
      MATERIALIEN EPR WQ P5 VON FT20 CARD XA
      100 100 21 21 121 124 4 1 1 1 4 2 0/ CARD XB
      0 0 0 0 0 0 20 0 0 0 0 0 0 / CARD XC
      19 A 3 1 ; FE
      1 -1 8.4600E-2 FE CARD XF
      ANALYSIS INPUT CARD AA
      9 100 121 0 0 2 0 0 1 1/ CARD BB
      VQ
      F 0. ; CARD CCX
      F 0. ; CARD CCY
      F 0. ; CARD CCY
      11 A 2 1 21 A 2 1 31 A 2 1 ;
      F 0 ;
      / NEUTRON FLUX [CM]
      R 100 1. /
      F 0. ;
      / GAMMA FLUX [CM]

```

CARD DD

R 100 0. /
F 1. ;

1 A 120 1 ;

CARD GG
CARD HH

1 / NUMBR
14 1. 0.0085 1 4.8295E-2 / NDETEC S KB NEL DENH
/ /ID = REGION NUMBERS
1001000 A 6 1000 2001000 A 6 1000;
F 6. ; ZZ
F 12. ; A
F 4.8295E-2 ; DEN
//G.FT32F001 DD *

	8	13	13	1000	1000	0	0
	4	0					
	0	1.000E+00					
PZ	1	0.000					
PZ	2	25.000					
PZ	3	50.000					
PZ	4	100.000					
CZ	11	0.000	0.000	0.000	2.000		
CZ	12	0.000	0.000	0.000	5.000		
CZ	13	0.000	0.000	0.000	10.000		
RCC	20	0.000	0.000	-1.000	0.000	0.000	102.000
		12.000					

END

1	11	11	1	-2	
1	12	11	2	-3	
1	13	11	3	-4	
1	21	12	-11	1	-2
1	22	12	-11	2	-3
1	23	12	-11	3	-4
1	31	13	-12	1	-2
1	32	13	-12	2	-3
1	33	13	-12	3	-4
1000	61	20	13	-1	
1000	62	20	13	4	
1000	63	20	-13		
-1	90	-20			

999

/* E O F

E.3.2 Output Files Excerpt for Sample Problem III

(1) Geometry Object File

	8	13	13	1000	1000	0	0
	4	0					
	0	1.000E+00					
PZ	1	0.000					
PZ	2	25.000					
PZ	3	50.000					
PZ	4	100.000					
CZ	11	0.000	0.000	0.000	2.000		
CZ	12	0.000	0.000	0.000	5.000		
CZ	13	0.000	0.000	0.000	10.000		
RCC	20	0.000	0.000	-1.000	0.000	0.000	102.000
		12.000					
END							
1	11	11	1	-2			
1	12	11	2	-3			
1	13	11	3	-4			
1	21	12	-11	1	-2		
1	22	12	-11	2	-3		
1	23	12	-11	3	-4		
1	31	13	-12	1	-2		
1	32	13	-12	2	-3		
1	33	13	-12	3	-4		
6666	61	20	13	-1			
6666	62	20	13	4			
6666	63	20	-13				
-1	90	-20					
999							

```

/JOB-ID = REM031H3 05/19/88 08:53:44
I:/ setup detector regions
I: INTEGER IRI(9);DATA IRI=<11 12 13 21 22 23 31 32 33>;
I:/ setup detector energy bins
I: REAL EP(12),EN(9);
I: DATA EP=< 1000. 750. 500. 250. 100. 50. 25. 20. 15. 10. 5. 1.>;
I: DATA EN=< 1000. 750. 500. 250. 100. 50. 25. 20. 15. >;
I:/ setup detectors
I: DEFDET PHIP,8,0.,,EP,,IRI;
I: DEFDET PHIN,8,1.,,EN,,IRI;
I:RUN; / return to het primary input
MAXBCH= 50 IBERTP=-20 EMIN(1)= 1.000E+00 ELOP = 2.000E+01
MAXCAS= 20 IOSUB = 21 EMIN(2)= 1.490E+01 CTOFE = 0.000E+00
IRUN = 1 NMSTP = 0 EPICUT = 0.000E+00 EMAX = 1.000E+03
EMUCUT = 0.000E+00 B0 = 8.000E+00
NICOL=0 MPIDK=1 NBOGUS=1 NSEUDO=1
NSPRD=0 NWSPRD=0 ISTRAG=0 IANG=1
IFISS=0 IBO=1 IGEOM=0 KEYDK=0
IELAS=2
ISEED= 123456789
LMI=0 MIA(5)= 0.0 MIZ(5)= 0.0 HIBE(5)= 0.000E+00
/BERTYP(1): I-N-C DATA READ FORM UNIT 20
/BERTYP(1): EVAP DATA READ FORM UNIT 20
NON-ISOTROPIC ANGLE OPTION SELECTED
MSIGMX 0.12000E-23 0.64942E-24 0.20000E-24 0.00000E+00 0.67700E-25
THIS IS MEDIUM 1
DENH=0.000000E+00 MEL= 1 SO= 1.000E+00 KB= 0.000E+00
ZZ( 1, 1)=26.0 A( 1, 1)= 56.0 DEN( 1, 1)=8.459997E-02
"GEOMETRIC" X-SECTIONS IN (1/CM)

NUCLIDS: 1 2 3 4 5 6 7 8 9 10
MEDIA
1 1.1592E-01
MAXIMUM M-1 X-SECTIONS IN (1/CM)

MEDIA PROTONS NEUTRONS PI+ P10 PI-
1 0.0000E+00 0.0000E+00 0.0000E+00 0.0000E+00 0.0000E+00
TRANSPORT X-SECTION = SUM("GEOMETRIC") + MAX. M-1 + MAX. DECAY X-SECTIONS IN (1/CM)

MEDIA PROTONS NEUTRONS PI+ P10 PI- MUE+ MUE-
1 1.1592E-01 1.1592E-01 1.2643E-01 0.0000E+00 1.2643E-01 1.1582E-04 1.1582E-04
SOURCE DATA:
TYP = 0.
XYZ = 0.000 0.000 -0.500 GAUSSIAN DISTRIBUTED WITH
XYZ SIG.= 0.000 0.000 0.000 TRUNCATED TO RADIUS
R = 0.000 IF R>0
UVW = 0.0000 0.0000 1.0000 IF = 0.,0.,0. -> ISOTROPIC
WT = 1.000
E = 1000.000 GAUSSIAN DISTRIBUTED WITH
E SIG.= 0.000
M:GOMIN INGE0=25 IOGEO= 6
NUMB. OF BODIES (NUMB) 8
NUMB. OF REGIONS (NUMR) 13
NUMBER OF CODE REGIONS (NCOD) 13
LENGTH OF FDP ARRAY (LMXX) 1000
LENGTH OF MA ARRAY (LMVY) 1000
MODIFIED INPUT (LOUT) 0
VOLUME INPUT OPTION (IVOPT) 4 NOT USED
BOUNDARY OPTION (MEDREG) 0
DEBUG OPTION (IDBG0) 0

MACHINE EPS = 0.100000E-07

PRINT= 0

```

STRETCH= 1.0000000E+00

NUMBER OF BODIES 8
LENGTH OF MA -ARRAY 57
LENGTH OF FPD-ARRAY 100
NUMBER OF INPUT ZONES 13
NUMBER OF CODE ZONES 13
LENGTH OF INTEGER ARRAY 238

--> SOR HET IRUN= 1 ISEED= 123456789
/SUBMIT(1): IOSUB=21
--> SOB HET IBAT= 1 ISEED= 123456789
/BERTTP(1): I-N-C DATA READ FORM UNIT 20
/DRES(1): USING 1977 WAPS DATA
--> EOB HET IBAT= 1 ISEED= 2139219343 NHIS= 20 CPU= 12.16SEC.
MEDIUM SORS CASC SLOW ESCP PSEU ABSO BRDY SCAT
1 0 109 130 0 61 3 135 0 0
6666 20 0 0 33 0 0 34 0 0
TOTAL 20 109 130 33 61 3 169 0 0
/SUBMIT(1): NEUTRONS: 130, PIO: 3, NUCLEI: 57

--> EOR HET IRUN= 1 ISEED= 1781818119 NHIS= 1000 CPU= 153.94SEC.
MEDIUM SORS CASC SLOW ESCP PSEU ABSO BRDY SCAT
1 0 6564 7001 0 3198 216 6740 0 0
6666 1000 0 0 1517 47 0 1523 0 0
TOTAL 1000 6564 7001 1517 3245 216 8263 0 0
REAL PSEUDO REAL PSEUDO PSEUDO REAL PSEUDO
NONHYDROG NONHYDROG HYDROGEN HYDROGEN DECAY ELASTIC ELASTIC
COLLISIONS COLLISIONS COLLISIONS COLLISIONS COLLISIONS COLLISIONS COLLISIONS
MEDIUM 1 0.35420E+04 0.27370E+04 0.00000E+00 0.00000E+00 0.11000E+02 0.32380E+04 0.45000E+03

PSEUDO COLLISIONS WITHIN INTERNAL VOID BY PARTICLES OF

TYPE 1 TYPE 2 TYPE 3 TYPE 4 TYPE 5 TYPE 6 TYPE 7
0.70000E+01 0.38000E+02 0.10000E+01 0.00000E+00 0.00000E+00 0.00000E+00 0.10000E+01
--> NESEX= 3 LOWAZ= 0 NWTC= 0
/SUBMIT(1): NEUTRONS: 7987, PIO: 184, NUCLEI: 3538

MEDIUM	SOURCE-FRACTION	%ERR		
1	0.000E+00	0.00		
VOID	1.000E+00	0.00		
MEDIUM	INELASTIC C./SOURCE	%ERR	ELASTIC C. /SOURCE	%ERR
1	3.326E+00	1.93	3.688E+00	2.52
VOID	0.000E+00	0.00	0.000E+00	0.00
MEDIUM	ABSORPTION /SOURCE	%ERR	FISSION /SOURCE	%ERR
1	2.160E-01	6.22	0.000E+00	0.00
VOID	0.000E+00	0.00	0.000E+00	0.00
MEDIUM	P FLUX (CM)	%ERR	N FLUX (CM)	%ERR
1	2.358E+01	2.01	2.940E+01	2.60
VOID	7.383E-01	4.64	3.977E+00	2.96
MEDIUM	PI+ FLUX (CM)	%ERR	PI- FLUX (CM)	%ERR
1	7.715E-01	10.77	2.502E-01	15.13
VOID	4.024E-02	27.05	1.456E-02	41.30
MEDIUM	MU+ FLUX (CM)	%ERR	MU- FLUX (CM)	%ERR
1	2.785E-03	15.35	1.021E-03	24.51
VOID	3.844E-04	26.50	7.651E-05	53.63
MEDIUM	P YIELD	%ERR	N YIELD	%ERR
1	8.662E+00	1.69	1.148E+01	1.94
VOID	0.000E+00	0.00	0.000E+00	0.00
MEDIUM	PI+ YIELD	%ERR	PI0 YIELD	%ERR
1	2.179E-01	7.14	1.839E-01	6.79
VOID	0.000E+00	0.00	0.000E+00	0.00
MEDIUM	PI- YIELD	%ERR		
1	1.059E-01	8.70		
VOID	0.000E+00	0.00		
MEDIUM	MU+ YIELD	%ERR	MU- YIELD	%ERR
1	7.825E-04	15.40	3.974E-04	20.98
VOID	3.346E-04	100.00	0.000E+00	0.00
MEDIUM	D YIELD	%ERR	T YIELD	%ERR
1	4.570E-01	5.84	1.110E-01	9.40
VOID	0.000E+00	0.00	0.000E+00	0.00
MEDIUM	HE3 YIELD	%ERR	HE4 YIELD	%ERR
1	1.010E-01	11.00	5.988E-01	3.82
VOID	0.000E+00	0.00	0.000E+00	0.00

W:RSYS: NO RSYST-TAPE AVAILABLE

FLUX DETECTOR FOR P

DETECTOR NAME = PHIP

REGION1=	11	REGION2=	0	
E-HIGH	E-LOW	PHI	%ERR	
1.000E+03	7.500E+02	1.135E+01	1.64	
7.500E+02	5.000E+02	1.982E+00	4.36	
5.000E+02	2.500E+02	7.813E-01	7.35	
2.500E+02	1.000E+02	9.172E-01	4.76	
1.000E+02	5.000E+01	4.632E-01	4.88	
5.000E+01	2.500E+01	2.360E-01	4.09	
2.500E+01	2.000E+01	4.486E-02	3.63	
2.000E+01	1.500E+01	4.322E-02	3.44	
1.500E+01	1.000E+01	4.320E-02	3.40	
1.000E+01	5.000E+00	4.284E-02	3.35	
5.000E+00	1.000E+00	2.607E-02	3.05	

FLUX DETECTOR FOR N

DETECTOR NAME = PHIN

REGION1=	11	REGION2=	0	
E-HIGH	E-LOW	PHI	%ERR	
1.000E+03	7.500E+02	1.907E-01	18.40	
7.500E+02	5.000E+02	3.181E-01	18.71	
5.000E+02	2.500E+02	9.127E-01	9.32	
2.500E+02	1.000E+02	1.463E+00	5.79	
1.000E+02	5.000E+01	1.148E+00	6.52	
5.000E+01	2.500E+01	1.094E+00	7.04	
2.500E+01	2.000E+01	3.437E-01	9.77	
2.000E+01	1.500E+01	5.862E-01	9.17	
REGION1=	12	REGION2=	0	
E-HIGH	E-LOW	PHI	%ERR	
1.000E+03	7.500E+02	6.895E-04	100.00	

```

-----
I:/ PHI.n(E,z,r) is available on PHIN@EOR
I:/ PHI.p(E,z,r) is available on PHIP@EOR
I:/ save detector output for later processing with CMD
I:/ TEST3 DETECTOR OUTPUT
I:SAVE PHIN@EOR;
I:SAVE PHINFSD;
I:SAVE PHIP@EOR;
I:SAVE PHIPFSD;
I:EXIT;

```


CMD Save File

```

REAL PHIN@EOR( 8, 1, 9);
DATA PHIN@EOR=<
1.907E-01 3.181E-01 9.127E-01 1.463E+00 1.148E+00
1.094E+00 3.437E-01 5.862E-01
6.895E-04 1.025E-01 2.712E-01 2.883E-01 2.637E-01
2.145E-01 5.589E-02 1.173E-01
0.000E+00 1.889E-02 2.115E-02 3.744E-02 1.482E-02
5.104E-02 3.063E-03 7.134E-03
1.075E-01 1.976E-01 6.795E-01 1.656E+00 1.431E+00
1.441E+00 4.577E-01 7.188E-01
4.842E-02 8.952E-02 3.477E-01 5.868E-01 5.670E-01
5.894E-01 1.559E-01 2.181E-01
0.000E+00 0.000E+00 8.035E-02 1.029E-01 6.867E-02
8.420E-02 9.328E-03 2.693E-02
1.419E-02 1.148E-01 5.668E-01 1.633E+00 1.595E+00
1.628E+00 5.130E-01 8.026E-01
7.823E-02 1.615E-01 4.979E-01 1.042E+00 7.951E-01
8.324E-01 2.383E-01 4.193E-01
1.126E-02 4.414E-02 1.499E-01 3.382E-01 3.578E-01
2.678E-01 4.400E-02 4.961E-02
>; /
REAL PHINFSD ( 8, 1, 9);
DATA PHINFSD =<
1.840E-01 1.871E-01 9.320E-02 5.792E-02 6.519E-02
7.037E-02 9.769E-02 9.167E-02
1.000E+00 3.463E-01 1.791E-01 1.394E-01 1.397E-01
1.589E-01 1.975E-01 1.516E-01
0.000E+00 1.000E+00 5.884E-01 3.802E-01 4.611E-01
3.870E-01 8.872E-01 6.789E-01
2.609E-01 1.876E-01 8.007E-02 6.117E-02 6.737E-02
5.774E-02 1.004E-01 7.584E-02
4.733E-01 3.009E-01 1.735E-01 8.848E-02 1.057E-01
1.177E-01 2.145E-01 1.503E-01
0.000E+00 0.000E+00 3.781E-01 2.614E-01 3.242E-01
3.128E-01 8.476E-01 4.033E-01
6.368E-01 2.554E-01 1.091E-01 7.556E-02 7.051E-02
6.410E-02 1.006E-01 8.117E-02
4.193E-01 2.317E-01 1.278E-01 1.006E-01 9.272E-02
1.147E-01 1.807E-01 1.377E-01
1.000E+00 4.840E-01 3.245E-01 2.143E-01 1.818E-01
2.043E-01 3.972E-01 3.087E-01
>; /
REAL PHIP@EOR( 11, 1, 9);
DATA PHIP@EOR=<
1.135E+01 1.982E+00 7.813E-01 9.172E-01 4.632E-01
2.360E-01 4.486E-02 4.322E-02 4.320E-02 4.284E-02
2.607E-02
0.000E+00 1.853E+00 1.141E+00 1.876E-01 1.008E-01
4.493E-02 8.279E-03 7.753E-03 7.390E-03 7.263E-03
4.394E-03
0.000E+00 0.000E+00 4.312E-02 2.223E-01 3.568E-02
1.058E-02 1.668E-03 1.472E-03 1.257E-03 1.066E-03
6.094E-04
3.556E-02 3.389E-01 4.187E-01 5.820E-01 2.235E-01
8.970E-02 1.455E-02 1.325E-02 1.185E-02 1.053E-02
6.164E-03
0.000E+00 1.454E-01 3.607E-01 2.168E-01 6.879E-02
2.887E-02 5.040E-03 4.631E-03 4.171E-03 3.773E-03
2.272E-03
0.000E+00 0.000E+00 6.810E-03 5.375E-02 1.332E-02
4.721E-03 7.377E-04 6.342E-04 5.368E-04 4.041E-04
2.472E-04
0.000E+00 7.748E-02 2.164E-01 2.076E-01 1.093E-01

```

```

4.920E-02 8.594E-03 7.891E-03 7.045E-03 6.537E-03
3.942E-03
0.000E+00 3.665E-02 2.063E-01 1.823E-01 7.429E-02
3.240E-02 5.486E-03 5.010E-03 4.636E-03 4.180E-03
2.604E-03
0.000E+00 0.000E+00 1.008E-02 2.425E-02 1.500E-02
6.728E-03 1.132E-03 1.026E-03 1.069E-03 1.008E-03
6.267E-04
> /
REAL PHIPFSD ( 11, 1, 9);
DATA PHIPFSD =<
1.638E-02 4.360E-02 7.345E-02 4.760E-02 4.876E-02
4.093E-02 3.628E-02 3.442E-02 3.400E-02 3.347E-02
3.050E-02
0.000E+00 6.682E-02 9.951E-02 1.011E-01 8.182E-02
8.936E-02 8.766E-02 8.599E-02 8.955E-02 9.200E-02
9.091E-02
0.000E+00 0.000E+00 1.642E-01 1.727E-01 1.862E-01
1.971E-01 1.817E-01 1.760E-01 1.716E-01 1.804E-01
1.841E-01
3.735E-01 1.538E-01 1.063E-01 7.755E-02 5.434E-02
5.564E-02 5.276E-02 5.195E-02 4.936E-02 4.954E-02
4.692E-02
0.000E+00 2.196E-01 1.565E-01 1.123E-01 9.868E-02
9.654E-02 8.567E-02 7.889E-02 7.782E-02 7.752E-02
7.246E-02
0.000E+00 0.000E+00 4.821E-01 2.958E-01 2.892E-01
2.618E-01 2.479E-01 2.470E-01 2.305E-01 2.126E-01
2.461E-01
0.000E+00 2.575E-01 1.329E-01 1.381E-01 9.393E-02
8.276E-02 7.787E-02 7.473E-02 6.978E-02 5.945E-02
5.997E-02
0.000E+00 3.864E-01 2.206E-01 1.355E-01 1.323E-01
1.177E-01 1.051E-01 1.052E-01 1.106E-01 1.008E-01
9.259E-02
0.000E+00 0.000E+00 1.000E+00 3.223E-01 2.225E-01
2.142E-01 2.087E-01 2.042E-01 1.891E-01 1.897E-01
1.878E-01
> /

```

(3) Output of MORSE

```

HERMES TEST3      9 DETEKTOREN      CARD A

HSTRT  NMOST  NITS  NQUIT  NGPQTH  NGPQTG  NMGP  NMTG  NCOLTP  IADJM  MAXTIM  MEDIA  MEDALB
  200    2500    50    1      100      21    100   121    0        0    58.00    1      0

ISOUR  NCPFS  ISBIAS      WTSTRT  EBOTN  EBOTG  TCUT  VELTH
  1      1      0      1.0000E+00  1.0000E-04  1.0000E+04  1.0000E-07  6.2930E+05

      XSTRT  YSTRT  ZSTRT  AGSTRT  UINP  VINP  WINP
      0.000E+00  0.000E+00  0.000E+00  0.000E+00  0.000000  0.000000  0.000000
***UNIT-NUMBER OF HERMES SUBMISSION : 50

GROUP PARAMETERS, GROUP NUMBERS GREATER THAN 100 CORRESPOND TO SECONDARY PARTICLES
GROUP  UPPER EDGE  VELOCITY
      (EV)      (CM/SEC)
  1      1.4900E+07  5.2115E+09
  2      1.3500E+07  4.9576E+09
  .
  .
  .
120      2.0000E+05  2.9979E+10
121      1.0000E+05  2.9979E+10

INITIAL RANDOM NUMBER = 35FA731A

NSPLY= 1  NKILL= 1  NPAST= 0  NOLEAK= 0  IEBIAS= 0  MXREG= 1  MAXGP=121

WEIGHT STANDARDS FOR SPLITTING AND RUSSIAN ROULETTE AND PATHLENGTH STRETCHING PARAMETERS
NCP1  NDC  NCP2  NRC1  NDRG  NRC2  WTHIN1  WTLWI  WTAVE1  XNU
  0    0    0    0    0    0  1.0000E+00  1.0000E+00  1.0000E+00  0.0000E+00

NSOUR= 0  NFISTP= 0  NKCALC= 0  NORMF= 0

GWLOW(IG,NREG)

GROUP  REGION 1
  1      1.0000E+00
  2      1.0000E+00
  .
  .
  .
 99      1.0000E+00
100      1.0000E+00

NUMB. OF BODIES (NUMB)      8
NUMB. OF REGIONS (NUMR)     13
NUMBER OF CODE REGIONS (NCOD) 13
LENGTH OF FDP ARRAY (LMXX)  1000
LENGTH OF MA ARRAY (LMYY)  1000
MODIFIED INPUT (LOUT)      0
VOLUME INPUT OPTION (IVOPT) 4 NOT USED
BOUNDARY OPTION (MEDREG)    0
DEBUG OPTION (IDBCO)        0

MACHINE EPS = 0.100000E-07

IPRINT= 0
STRETCH= 1.0000000E+00

NUMBER OF BODIES      8
LENGTH OF MA -ARRAY   57
LENGTH OF FDP-ARRAY   100
NUMBER OF INPUT ZONES  13
NUMBER OF CODE ZONES   13

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MATERIALIEN EPR WQ P5 VON FT20

CARD XA

NUMBER OF PRIMARY GROUPS (NGP) 100
 NUMBER OF PRIMARY DOWNSCATTERS (NDS) 100
 NUMBER OF SECONDARY GROUPS (NGG) 21
 NUMBER OF SECONDARY DOWNSCATTERS (NDSC) 21
 NUMBER OF PRIM+SEC GROUPS (INCP) 121
 TABLE LENGTH (ITBL) 124
 LOC OF WITHIN GROUP (SIG GG) (ISCG) 4
 NUMBER OF MEDIA (NMED) 1
 NUMBER OF INPUT ELEMENTS (NELEM) 1
 NUMBER OF MIXING ENTRIES (NMIX) 1
 NUMBER OF COEFFICIENTS (NCOEF) 4
 NUMBER OF ANGLES (NSCT) 2
 RESTORE COEFF (ISTAT) 0
 ADJOINT SWITCH (FROM MORSE) 0

INPUT/OUTPUT OPTIONS

IRDSG (AS READ) 0
 ISTR (AS STORE) 0
 IFMU (MUS) 0
 IMOM (MOMENTS) 0
 IPRIN (ANGLES, PROB) 0
 IPUN (IMPOSSIBLE COEF) 0
 CARD FORMAT (IDTF) 0
 INPUT TAPE (IXTAPE) 20
 MORSEC TAPE (JXTAPE) 0
 OGR TAPE (IOGRT) 0
 ICQPT (LAST GRP. OGR) 0
 CROSS-TAPE AUS RSYST 0
 ICRO CROSS. FOR MEDIA
 N TO G TRANSFER 0
 PRINT OF BADMOMENTS 0

ELEMENTS FROM LIBRARY TAPE

IDENTIFIERS 19 20 21 22
 ELEMENT 1 19

121 124 0 19FE 1192 100N-21G GPS P8 800K INF DIL

07/25/84

STORAGE ALLOCATIONS

CROSS SECTIONS START AT 5186
 LAST LOCATION USED (PERM) 18995
 TEMP LOCATIONS USED 928664 TO 950000
 EXCESS STORAGE (TEMP) 909669

ELEMENT 1 19 121 124 0 19FE 1192 100N-21G GPS P8 800K INF DIL
 ELEMENT 1 20 121 124 0 20FE 1192 100N-21G GPS P8 800K INF DIL
 ELEMENT 1 21 121 124 0 21FE 1192 100N-21G GPS P8 800K INF DIL
 ELEMENT 1 22 121 124 0 22FE 1192 100N-21G GPS P8 800K INF DIL

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07/25/84

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MIXING TABLE

MEDIA 1 CONTAINS ELEMENT 1 WITH DENSITY 8.4600E-02
 BANKS START AT 18996
 LAST LOCATION USED 56495

ANALYSIS INPUT
 ND= 9, NNE=100, NE=121, NT= 0, NA= 0, NRESP= 2, NEX= 0, NEXND= 0
 IORESP=1 IOFLUX=1
 DET X Y Z MED1 MED2

TO IS SET TO 0.0 IN ALL CASES

1	11	0.0000E+00	0.0000E+00
2	12	0.0000E+00	0.0000E+00
3	13	0.0000E+00	0.0000E+00
4	21	0.0000E+00	0.0000E+00
5	22	0.0000E+00	0.0000E+00
6	23	0.0000E+00	0.0000E+00

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7				31	0.0000E+00	0.0000E+00
8				32	0.0000E+00	0.0000E+00
9				33	0.0000E+00	0.0000E+00
GROUP	RESP(1)	RESP(2)				
1	1.0000E+00	0.0000E+00				
2	1.0000E+00	0.0000E+00				
	.					
	.					
120	0.0000E+00	1.0000E+00				
121	0.0000E+00	1.0000E+00				
	NUMBER OF PRIMARY ENERGY BINS	100				
	TOTAL NUMBER OF ENERGY BINS	121				
BIN NO.	LOWER LIMIT GROUP	LOWER ENERGY LIMIT	DELTA E			
		1.490E+07				
1	1	1.350E+07	1.400E+06			
2	2	1.220E+07	1.300E+06			
	.					
	.					
120	120	1.000E+05	1.000E+05			
121	121	1.000E+04	9.000E+04			
	NUMBER OF TIME BINS	0				
	NUMBER OF ANGLE BINS	0				
	UPPER LIMITS OF COSINE BINS					

4295 CELLS USED BY ANALYSIS, 889210 CELLS REMAIN UNUSED.

TIME REQUIRED FOR INPUT WAS 4 SECONDS .
YOU ARE USING THE DEFAULT VERSION OF STRUM WHICH DOES NOTHING.

***START BATCH 1 RANDOM= 35FA731A

SOURCE DATA

MET-IDENTIFIKATION: REWO 31H3 05/1 9/88 08:5 3:44

YOU ARE USING THE DEFAULT VERSION OF SOURCE WHICH SETS WATE TO DOF AND PROVIDES AN ENERGY IC.

WTAVE	IAVE	UAVE	VAVE	WAVE	XAVE	YAVE	ZAVE	AGEAVE
1.300E+02	16.34	-0.0529	-0.0522	0.0723	2.100E-01	1.443E-02	1.129E+01	0.000E+00

NUMBER OF COLLISIONS OF TYPE NCOLL		SOURCE SPLIT(D)		FISHN	GAMGEN	REALCOLL	ALBEDO	BDRYX	ESCAPE	E-CUT	TIMEKILL	R R	KILL R R	SURV	GAMLOST	FISGEN
130	67	0	176	1429	0	631	165	0	6	205	1231	0	0	0	0	0

TIME REQUIRED FOR THE PRECEDING BATCH WAS 1 SECOND .

***START BATCH 2 RANDOM= 788D386A

SOURCE DATA

WTAVE	IAVE	UAVE	VAVE	WAVE	XAVE	YAVE	ZAVE	AGEAVE
2.100E+02	17.93	0.0531	0.0334	0.0817	6.616E-01	-7.574E-02	1.764E+01	0.000E+00

NUMBER OF COLLISIONS OF TYPE NCOLL		SOURCE SPLIT(D)		FISHN	GAMGEN	REALCOLL	ALBEDO	BDRYX	ESCAPE	E-CUT	TIMEKILL	R R	KILL R R	SURV	GAMLOST	FISGEN
210	114	0	311	2532	0	1059	268	0	6	363	2173	0	0	0	0	0

TIME REQUIRED FOR THE PRECEDING BATCH WAS 2 SECONDS .

***START BATCH 49 RANDOM= 2F2E804E

SOURCE DATA

WTAVE	IAVE	UAVE	VAVE	WAVE	XAVE	YAVE	ZAVE	AGEAVE
1.740E+02	19.09	-0.0003	0.0174	0.0584	-2.648E-01	-2.026E-01	1.019E+01	0.000E+00

NUMBER OF COLLISIONS OF TYPE NCOLL		SOURCE SPLIT(D)		FISHN	GAMGEN	REALCOLL	ALBEDO	BDRYX	ESCAPE	E-CUT	TIMEKILL	R R	KILL R R	SURV	GAMLOST	FISGEN
174	96	0	246	2145	0	906	228	0	6	282	1872	0	0	0	0	0

TIME REQUIRED FOR THE PRECEDING BATCH WAS 2 SECONDS .

***START BATCH 50 RANDOM= 33AAFE8A

SOURCE DATA

WTAVE	IAVE	UAVE	VAVE	WAVE	XAVE	YAVE	ZAVE	AGEAVE
1.510E+02	18.46	0.0260	-0.0326	0.1471	1.675E-01	7.725E-03	1.521E+01	0.000E+00

NUMBER OF COLLISIONS OF TYPE NCOLL		SOURCE SPLIT(D)		FISHN	GAMGEN	REALCOLL	ALBEDO	BDRYX	ESCAPE	E-CUT	TIMEKILL	R R	KILL R R	SURV	GAMLOST	FISGEN
151	80	0	205	1824	0	786	199	0	5	233	1593	0	0	0	0	0

TIME REQUIRED FOR THE PRECEDING BATCH WAS 1 SECOND .

/ NEUTRON FLUX [CM]

DETECTOR	RESPONSES(DETECTOR)		TOTAL RESPONSE	FSD TOTAL
	UNCOLL RESPONSE	FSD UNCOLL		
1	0.0000E+00	0.00000	1.7990E+01	0.02723
2	0.0000E+00	0.00000	4.6661E+00	0.07635
3	0.0000E+00	0.00000	5.1120E-01	0.18050
4	0.0000E+00	0.00000	4.0528E+01	0.02972
5	0.0000E+00	0.00000	1.3825E+01	0.04608
6	0.0000E+00	0.00000	1.7700E+00	0.13744
7	0.0000E+00	0.00000	5.7800E+01	0.03056
8	0.0000E+00	0.00000	2.3853E+01	0.04504
9	0.0000E+00	0.00000	3.8090E+00	0.10494

CARD DD

/ GAMMA FLUX [CM]

DETECTOR	RESPONSES(DETECTOR)		TOTAL RESPONSE	FSD TOTAL
	UNCOLL RESPONSE	FSD UNCOLL		
1	0.0000E+00	0.00000	6.9535E+00	0.04039
2	0.0000E+00	0.00000	1.7514E+00	0.08446
3	0.0000E+00	0.00000	1.6529E-01	0.24773
4	0.0000E+00	0.00000	1.8049E+01	0.03801
5	0.0000E+00	0.00000	5.0836E+00	0.06550
6	0.0000E+00	0.00000	5.4571E-01	0.13158
7	0.0000E+00	0.00000	2.1992E+01	0.04181
8	0.0000E+00	0.00000	7.2519E+00	0.05949
9	0.0000E+00	0.00000	1.2392E+00	0.13384

CARD DD

DETECTOR NO. ENERGIES	1	2	3	4	5	6	7	8	9
1.490E+07	1.676E-07 0.161	2.924E-08 0.341	0.000E+00 0.000	1.674E-07 0.152	4.382E-08 0.349	0.000E+00 0.000	1.608E-07 0.151	4.791E-08 0.351	3.260E-08 0.425
1.350E+07	1.965E-07 0.110	4.248E-08 0.332	1.294E-09 1.000	2.544E-07 0.116	5.971E-08 0.222	1.129E-09 1.000	2.733E-07 0.144	9.444E-08 0.229	3.221E-09 0.616
1.220E+07	1.642E-07 0.133	1.928E-08 0.278	1.216E-09 1.000	2.143E-07 0.151	5.141E-08 0.320	5.498E-09 0.987	2.425E-07 0.161	8.117E-08 0.204	9.896E-09 0.579

CARD GG

DETECTOR NO. ENERGIES	1	2	3	4	5	6	7	8	9
2.000E+05	7.954E-06 0.049	2.334E-06 0.119	2.676E-07 0.328	2.436E-05 0.046	7.016E-06 0.072	6.356E-07 0.178	2.892E-05 0.049	1.026E-05 0.073	1.497E-06 0.140
1.000E+05	2.826E-08 0.097	7.255E-09 0.210	6.607E-10 0.565	8.358E-08 0.059	2.239E-08 0.100	2.931E-09 0.329	9.773E-08 0.050	3.293E-08 0.129	3.607E-09 0.210
1.000E+04									

CARD GG

TIME REQUIRED FOR THE PRECEDING 50 BATCHES WAS 1 MINUTE 17 SECONDS .

